



TRILATERAL
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531st Wilhelm and Else Heraeus Seminar

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Active Control of Tokamak Edge Instabilities by 3D Fields

Yunfeng Liang

Forschungszentrum Jülich GmbH, IEK-4, 52425 Jülich, Germany



- Introduction to ELMs and 3D fields on tokamak
- Summary of ELM control with 3D fields
 - ELM suppression
 - ELM mitigation
- What are the possible physical mechanisms of ELM control with 3D fields?
 - Resistive Plasma Responses
 - Field Penetration / Mode Excitation
 - Edge Ergodisation
 - Ideal Plasma Responses
 - Rotation Screening Effect
 - Resonant Field Amplification (RFA)
- ELM control through the magnetic topology control by another methods
- Summary

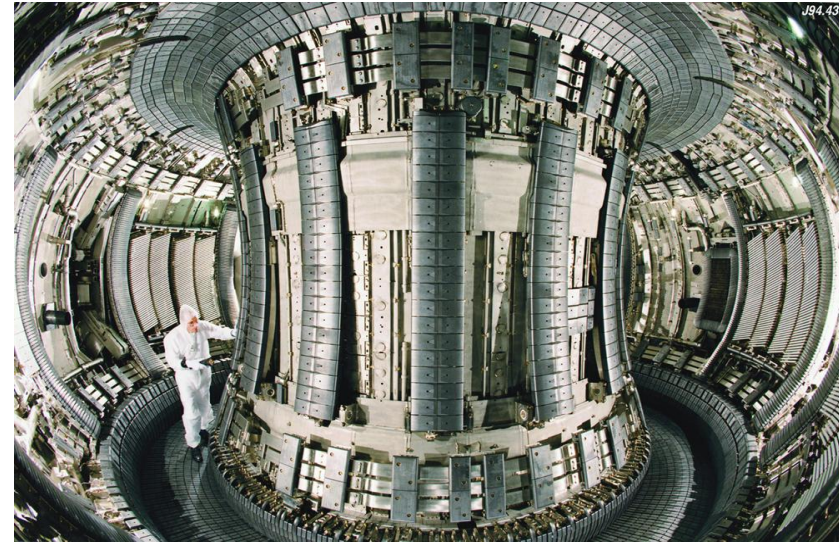


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Fusion ...

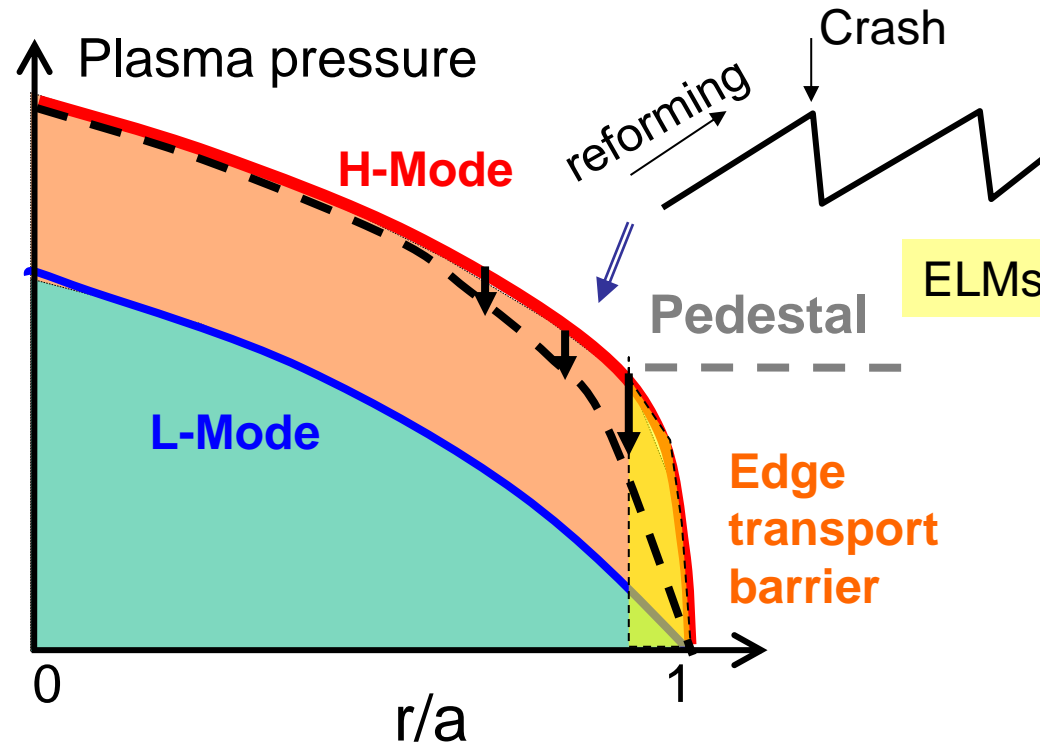


... on Earth



- A solar flare is a large explosion in the Sun's atmosphere that can release as much as 6×10^{25} joules of energy (~ 17% of the total energy output of the Sun each second)
- A 'natural' Edge Localized Mode (ELM) size in ITER:

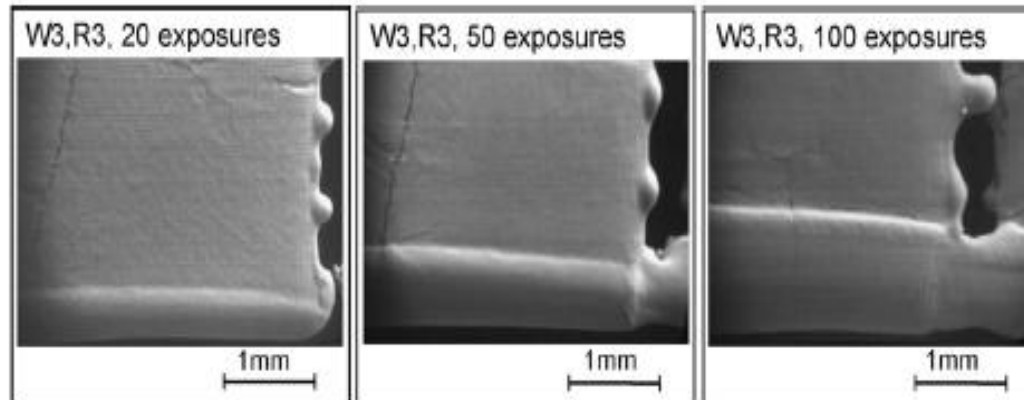
$$\Delta W_{\text{ELM}}, \sim 20 \text{ MJ} \rightarrow 10 \text{ MJ/m}^2$$



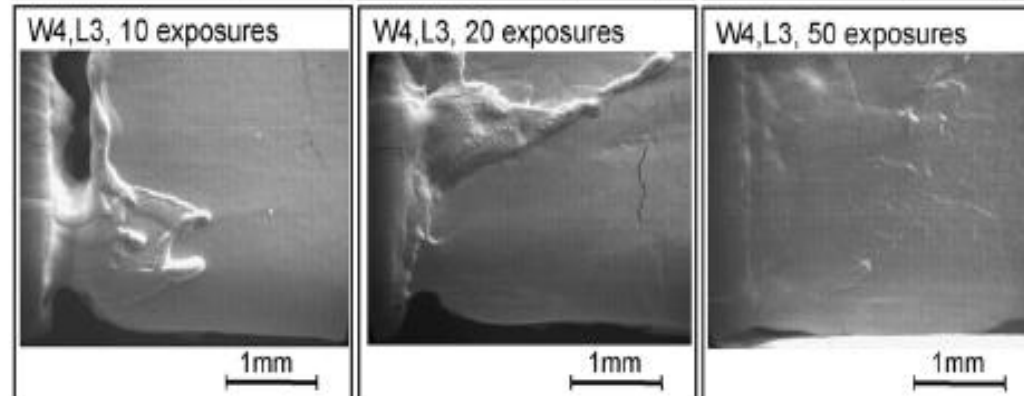
Large ELM $\rightarrow \Delta W_p / W_p$ up to 20%

ELM control is important for both tokamak and stellarator H-mode plasmas

$Q = 1.0 \text{ MJ/m}^2$



$Q = 1.6 \text{ MJ/m}^2$



Zhitlukhin JNM 2007

- $0.4\text{-}1.0 \text{ MJ/m}^2$ (JET < 1.0 MJ/m^2) → Edge melting and surface cracking
- $1.0\text{-}1.6 \text{ MJ/m}^2$ → Surface melting, bridge formation and droplet ejection
- High frequency ELMs may be required to avoid W accumulation

- It is widely believed that ideal MHD instabilities provide the trigger for the ELM
- Theoretically, the instability properties can be understood from δW for radial displacement, X , at large toroidal mode number, n :

$$\delta W = \pi \int_0^{\psi_a} d\psi \oint d\theta \left\{ \frac{JB^2}{R^2 B_p^2} |k_{\parallel} X|^2 + \frac{R^2 B_p^2}{JB^2} \left| \frac{1}{n} \frac{\partial}{\partial \psi} (JBk_{\parallel} X) \right|^2 \right.$$

Field-line bending:
strongly stabilising unless
 k_{\parallel} is small

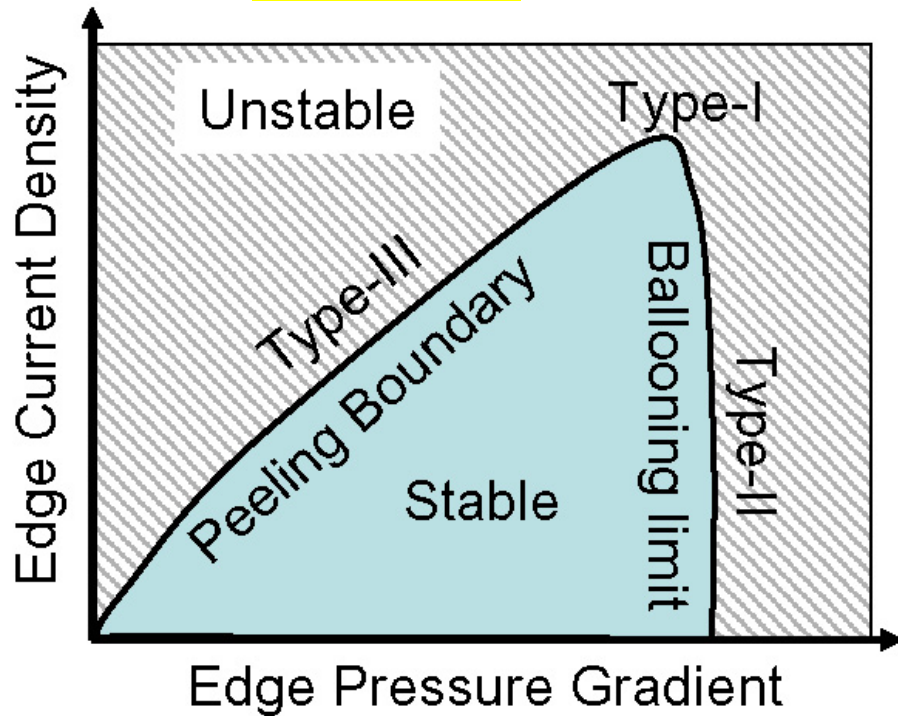
$$\left. - \frac{2J}{B^2} \frac{dp}{d\psi} \left[|X|^2 \frac{\partial}{\partial \psi} \left(p + \frac{B^2}{2} \right) - \frac{i}{2} \frac{f}{JB^2} \frac{\partial B^2}{\partial \theta} \frac{X^*}{n} \frac{\partial X}{\partial \psi} \right] \right.$$

Pressure gradient/curvature
drive: destabilising if average
curvature is “bad”

$$\left. - \frac{X^*}{n} JBk_{\parallel} \left(\frac{\partial \sigma}{\partial \psi} X \right) + \frac{\partial}{\partial \psi} \left[\frac{\sigma}{n} X JBk_{\parallel}^* X^* \right] \right.$$

Current density gradient/edge
current drives kink/peeling
modes
 σ =normalised current density

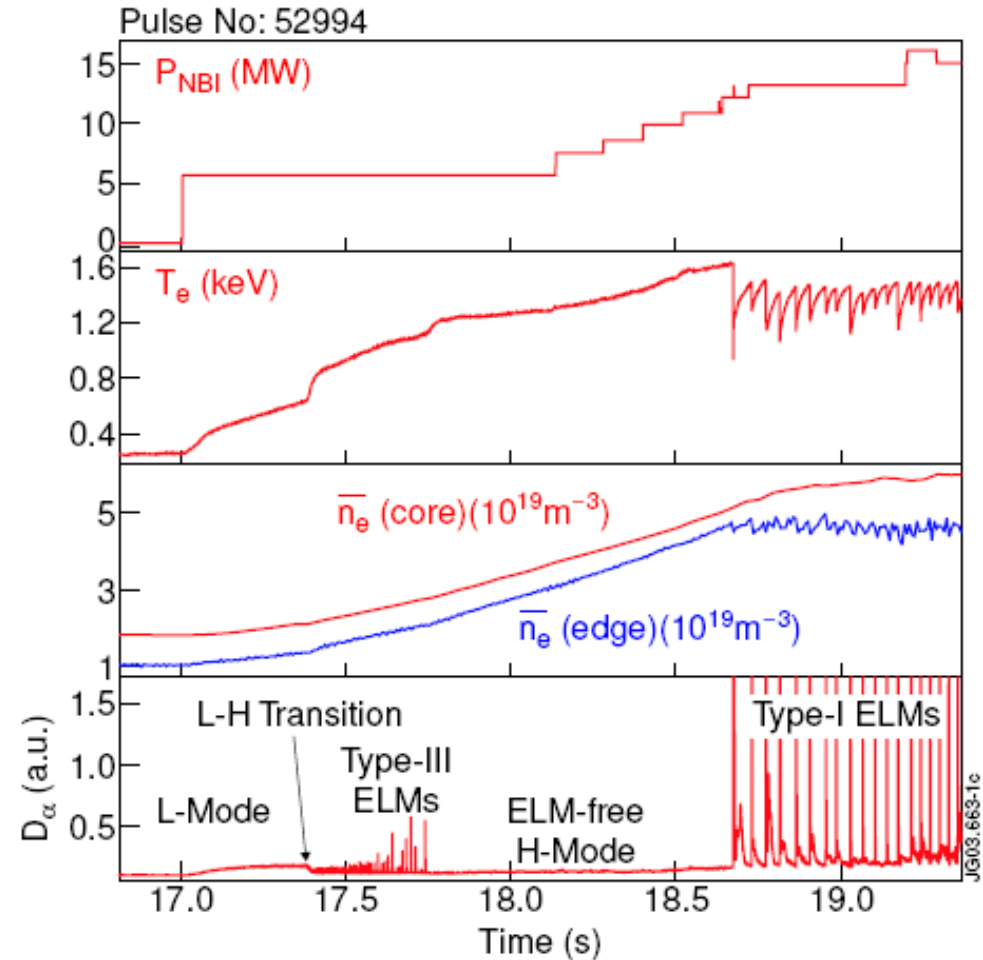
Tokamak



Stellarator

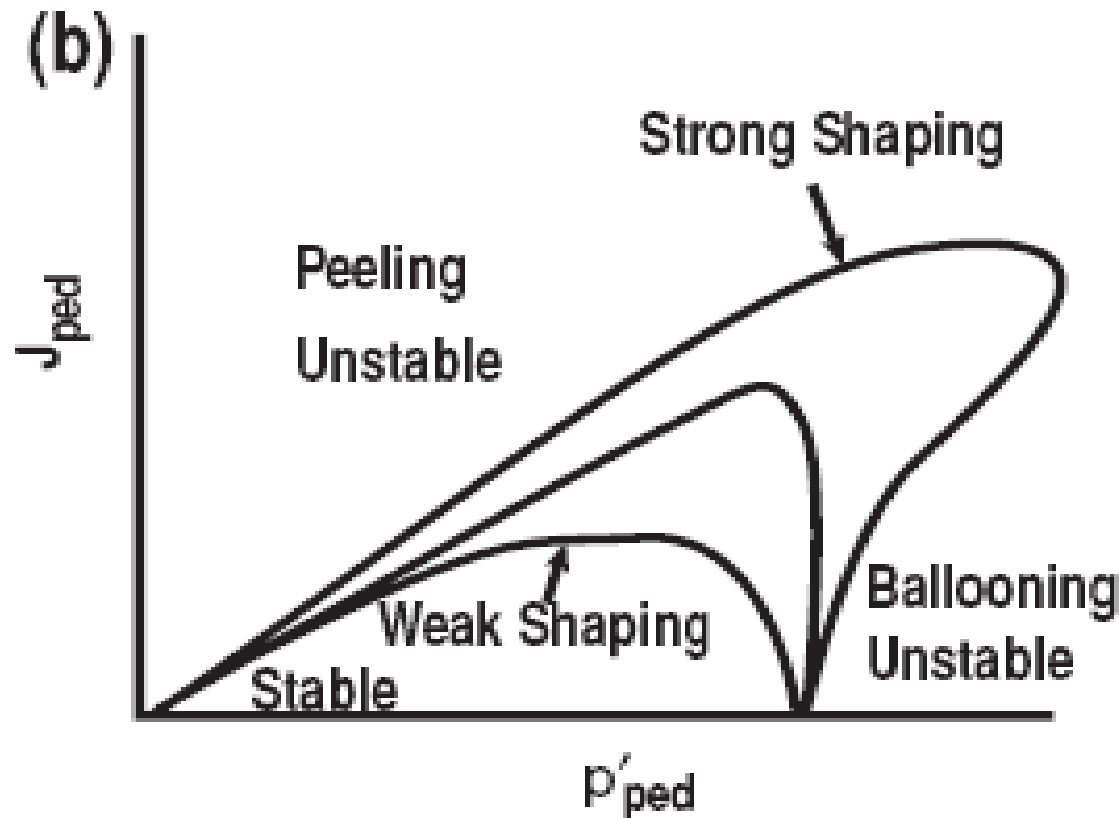
Edge region in magnetic hill

→ Resistive Interchange mode (RIC) are unstable.

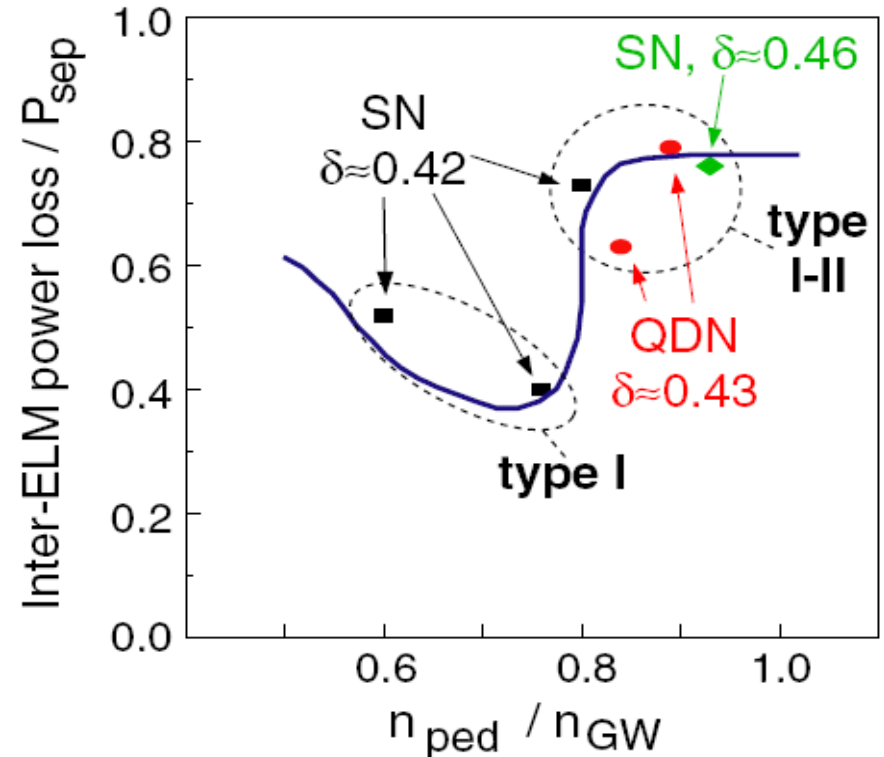
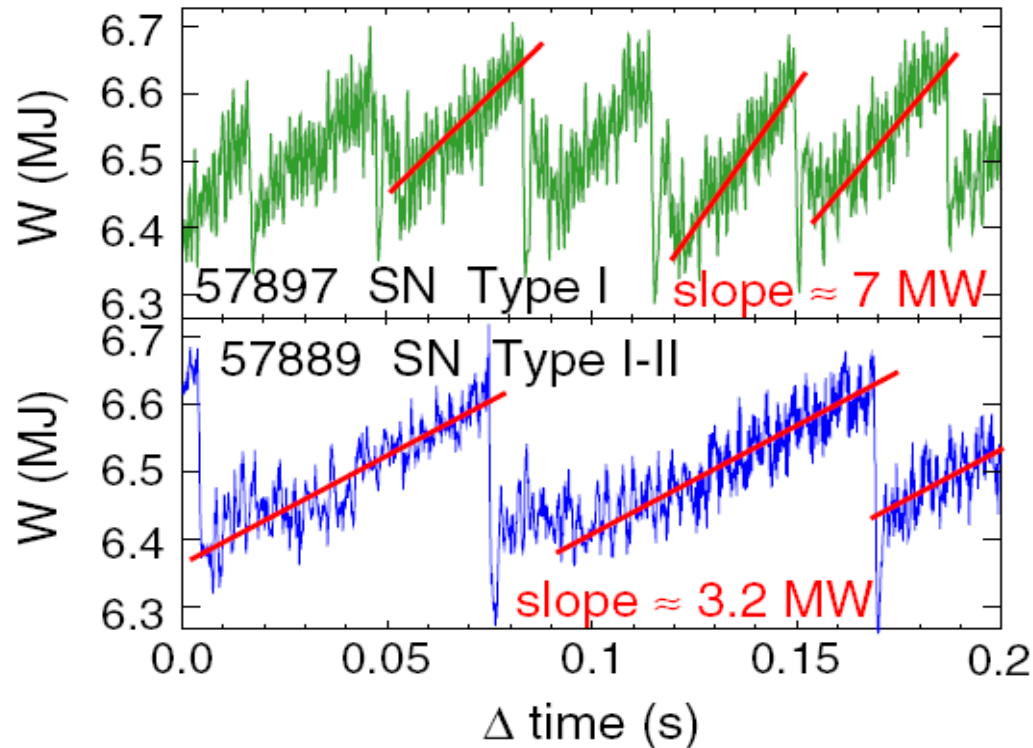


Bhatnagar V P *et al* 1999 *Nucl. Fusion* **39** 353

Variation of Pedestal Stability Boundaries with Plasma Shaping



P.B. Snyder *et al*, Nucl. Fusion **44** (2004) 320



J. Stober, et al., Nuclear Fusion, 45,1213 (2005)

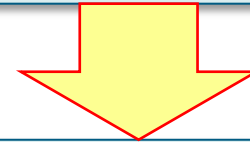
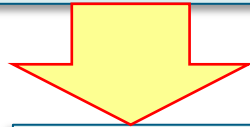
Mixed Type I and II ELM H-mode has been observed in high δ and high density plasmas in JET



Theoretical Prediction

Pedestal pressure
Edge current density

Plasma configurations
Plasma shaping



ELM control

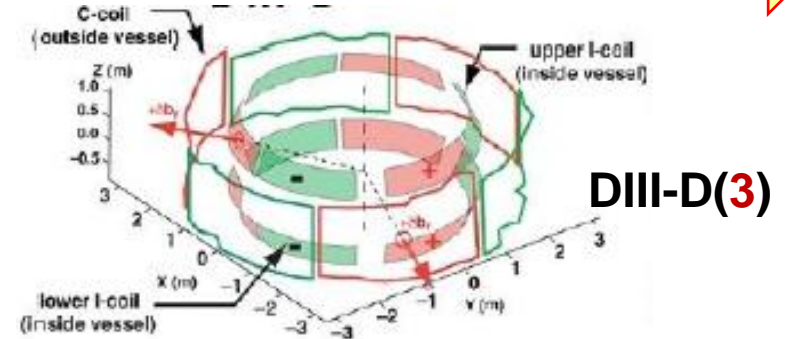
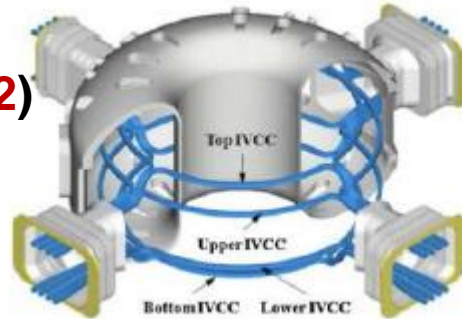


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Toroidal mode number

ELM suppression

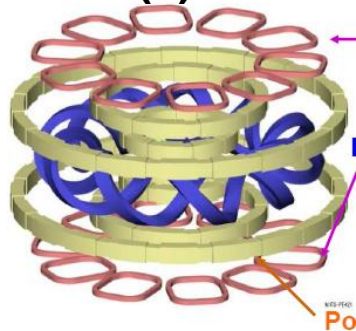
KSTAR(2)



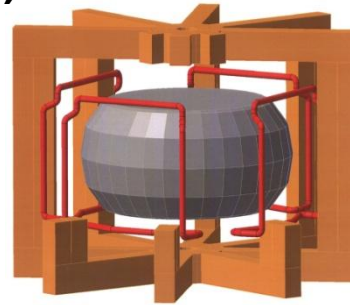
ELM Mitigation

External coils

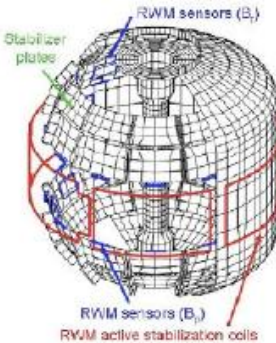
LHD(1)



JET(2)

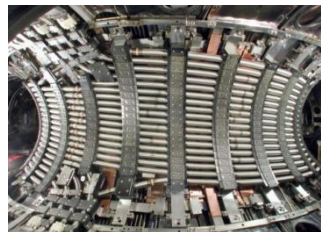


NSTX(3)

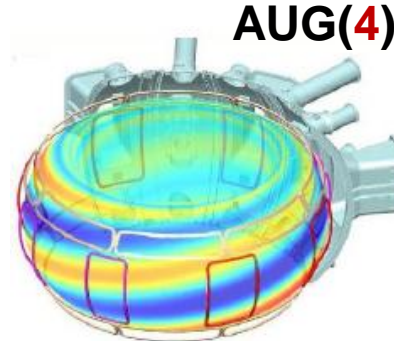


In-vessel coils

COMPASS-D (1)



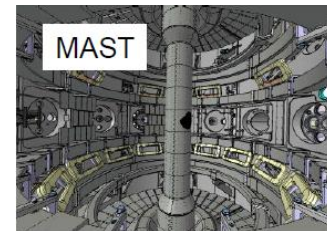
TEXTOR(4)

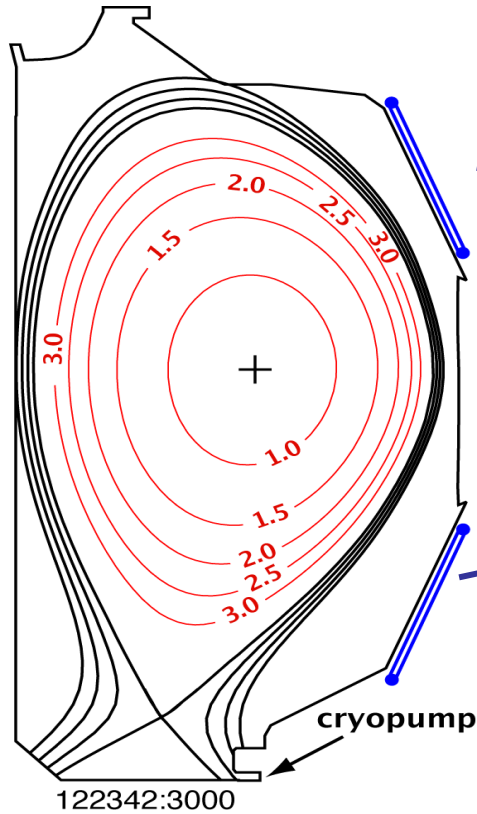


AUG(4)

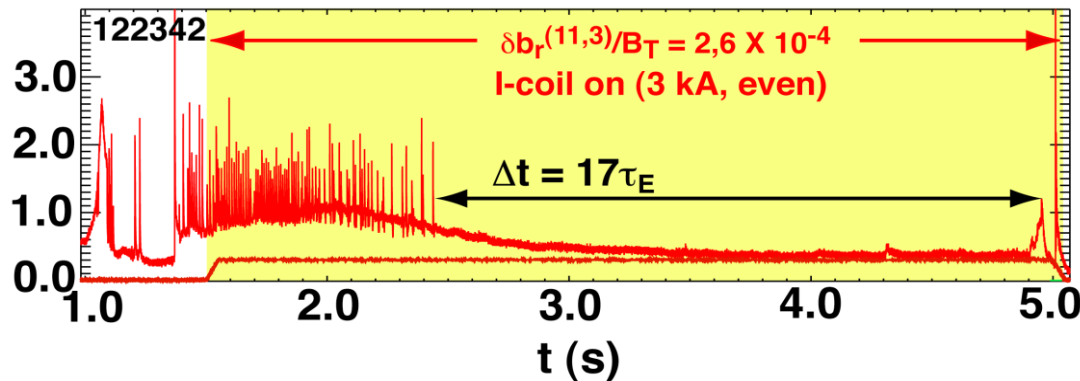
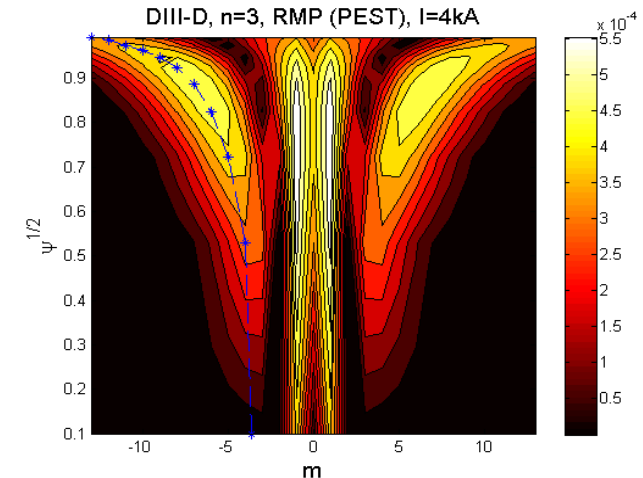
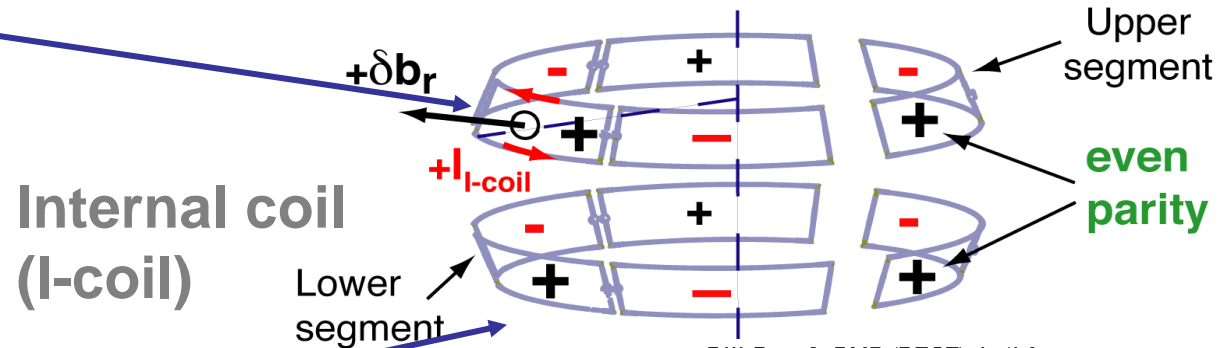
JFT-2M (>4)

MAST(6)

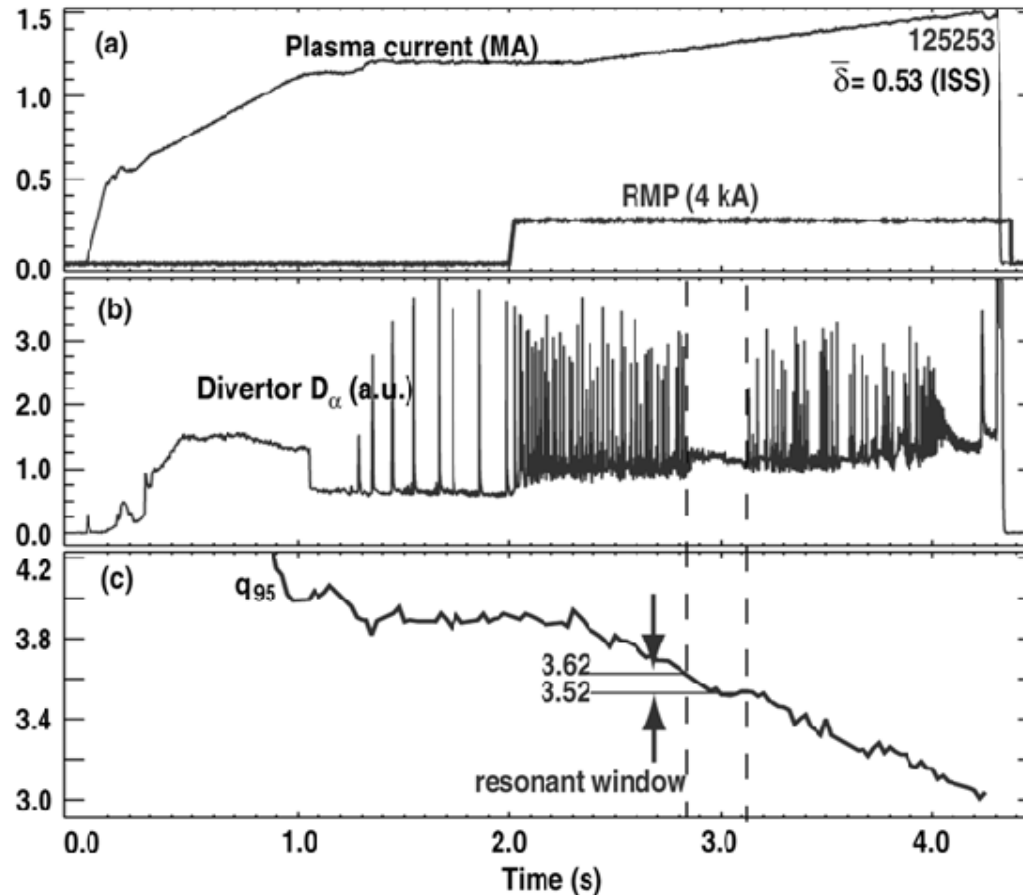




**n=3 I-coil configuration
(strong RMP - even parity)**



- T. E. Evans, et al., PRL, 92, 235003 (2004)*
- T. E. Evans, et al., Nature physics, Vol. 2, p419, June 2006*
- T. E. Evans, et al., Phys. Plasmas 13, 056121 (2006).*



T.E. Evans, et al.,
 NF 48 (2008) 024002

- ✓ ELM suppression achieved in a narrow q_{95} window on DIII-D with an $n=3$ field induced by the I-coils.
- ✓ q_{95} ELM suppression window can be enlarged slightly with a mixed $n=1$ and $n=3$ fields.

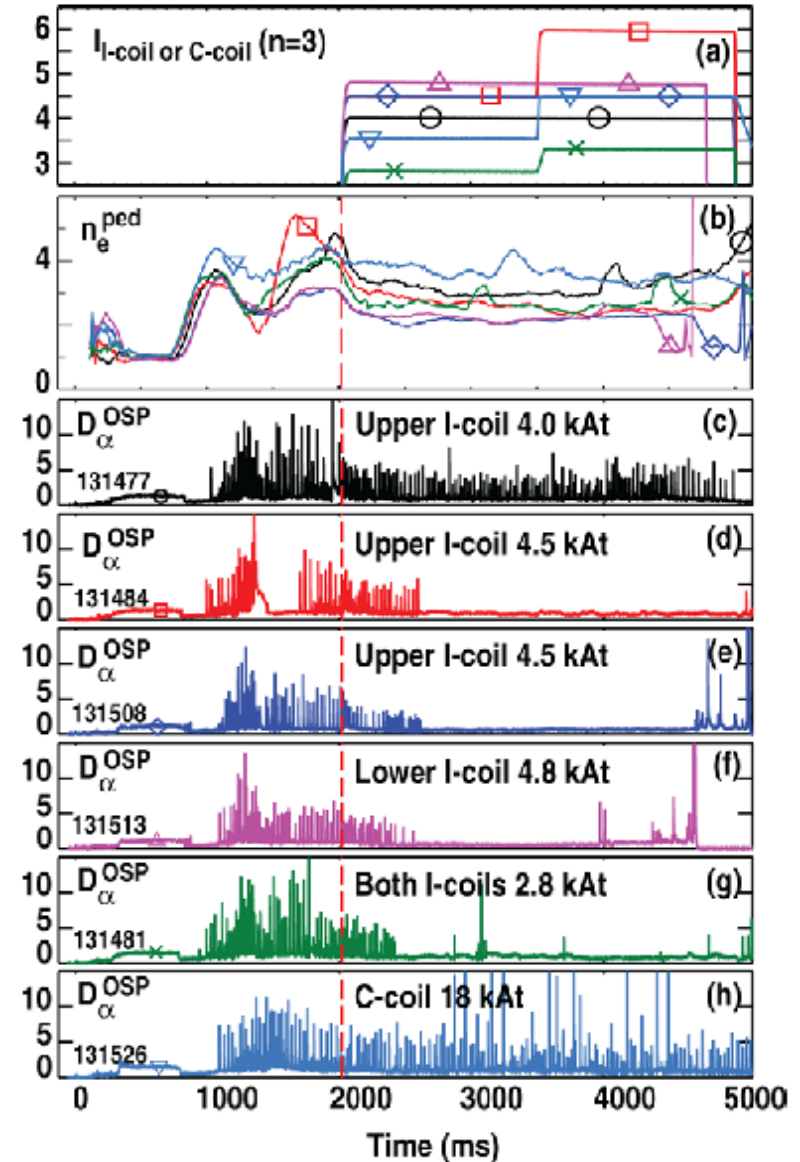
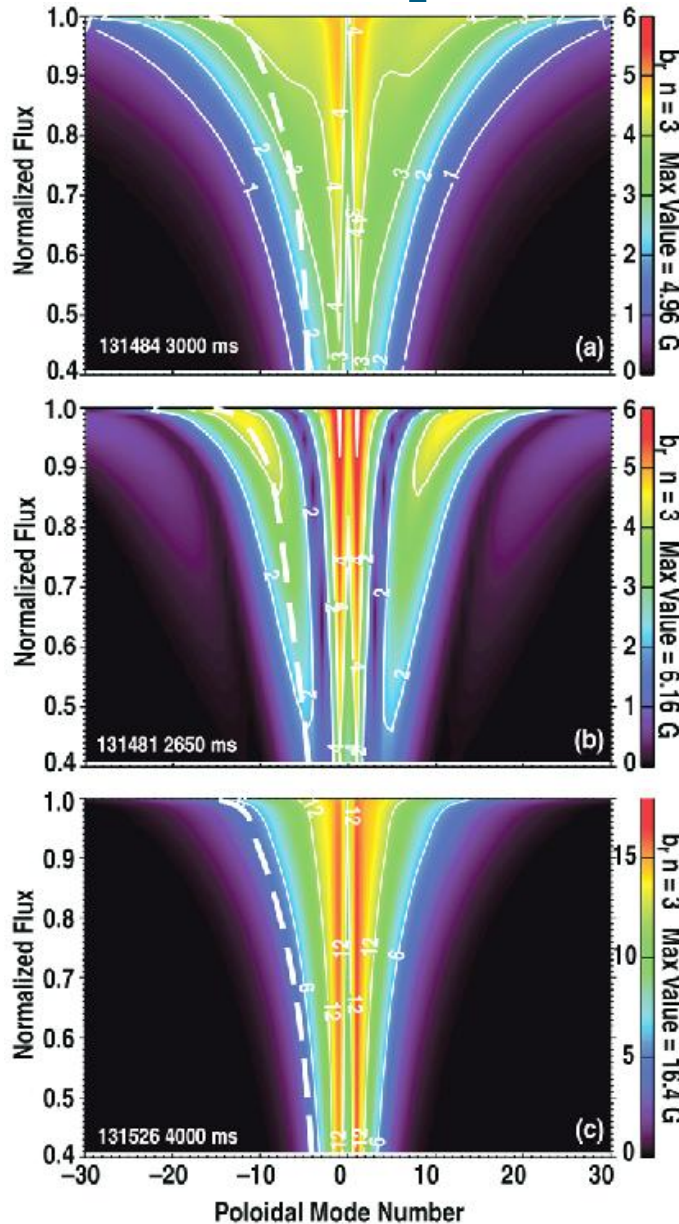
What is the role of the magnetic perturbation spectrum?

DIII-D

Upper in-vessel coils only

Both Upper and lower In-vessel coils

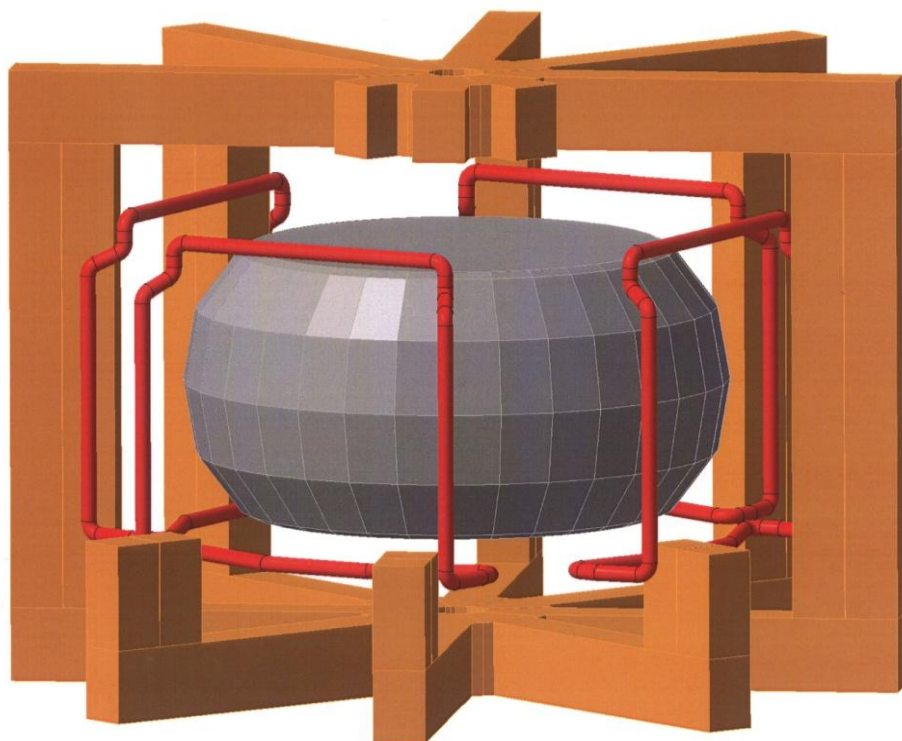
External C coils



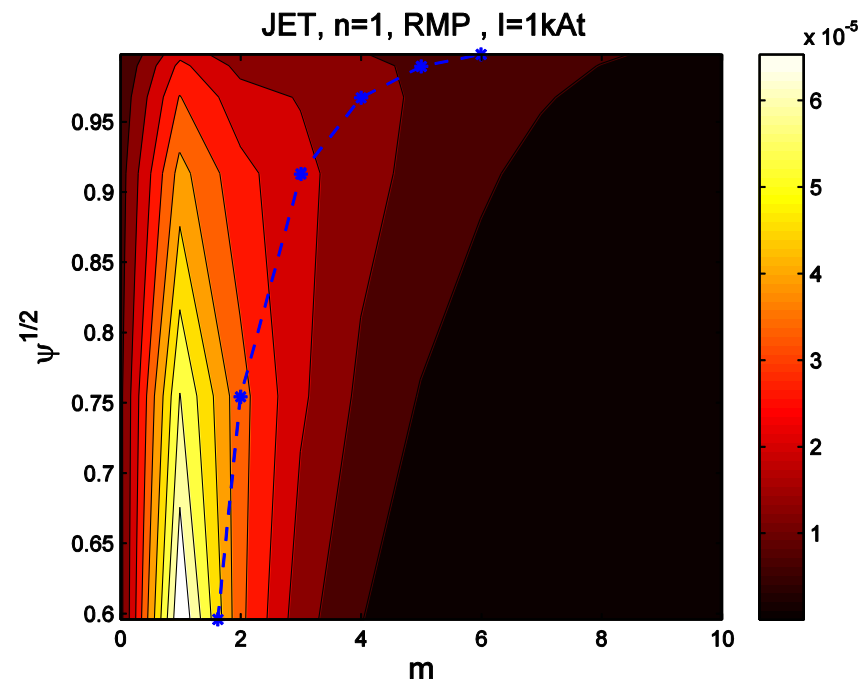
M.E. Fenstermacher, NF (2008)



Error Field Correct Coils (EFCC) on JET



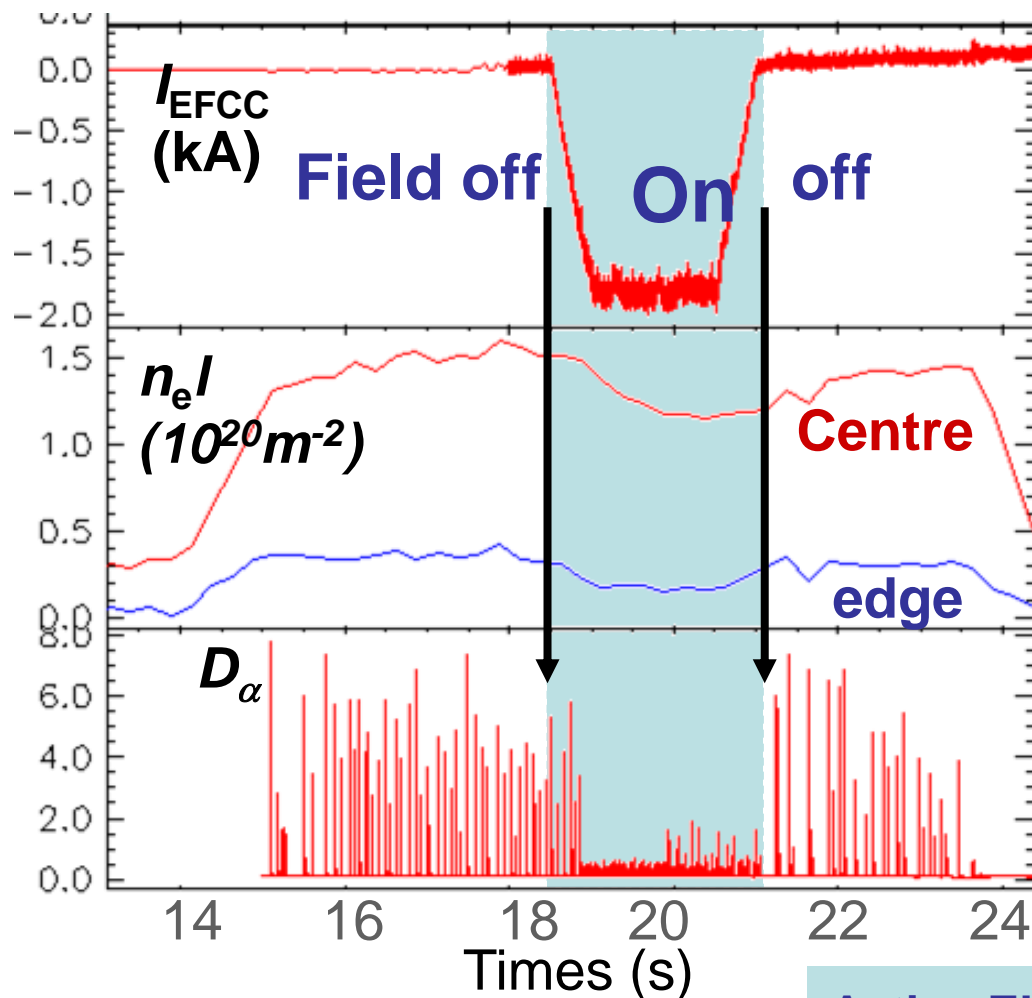
$$I_{\text{EFCC}} = 1 \text{ kAt}; B_t = 1.84 \text{ T}$$



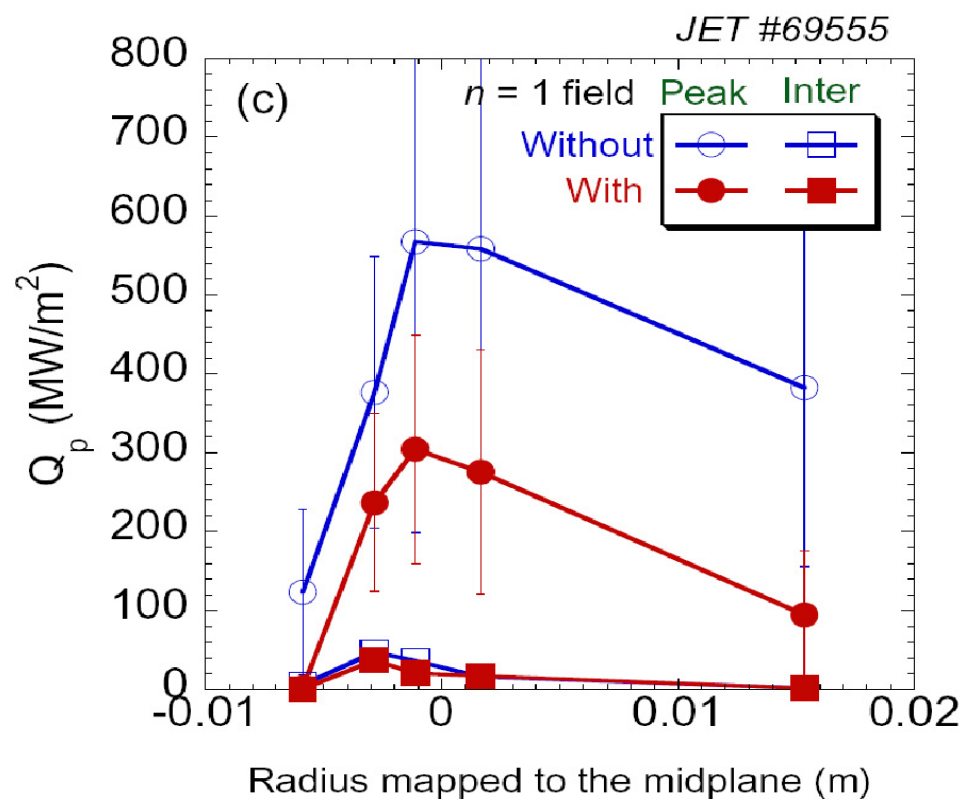
- Depending on the relative phasing of the currents in individual coils, either $n=1$ or $n=2$ fields can be generated
- $I_{\text{Coil}} \leq 3$ (6) kA x 16 turns ($n = 1$ and 2)
- $R \sim 6$ m; Size ~ 6 m * 6 m
- B_r at wall ~ 0.25 mT/kAt

Y.Liang et al., PPCF 2007

$I_p = 1.8$ MA; $B_t = 2.1$ T; $q_{95} \sim 4.0$; $\delta_U \sim 0.45$ JET#69557



Heat flux onto the outer divertor

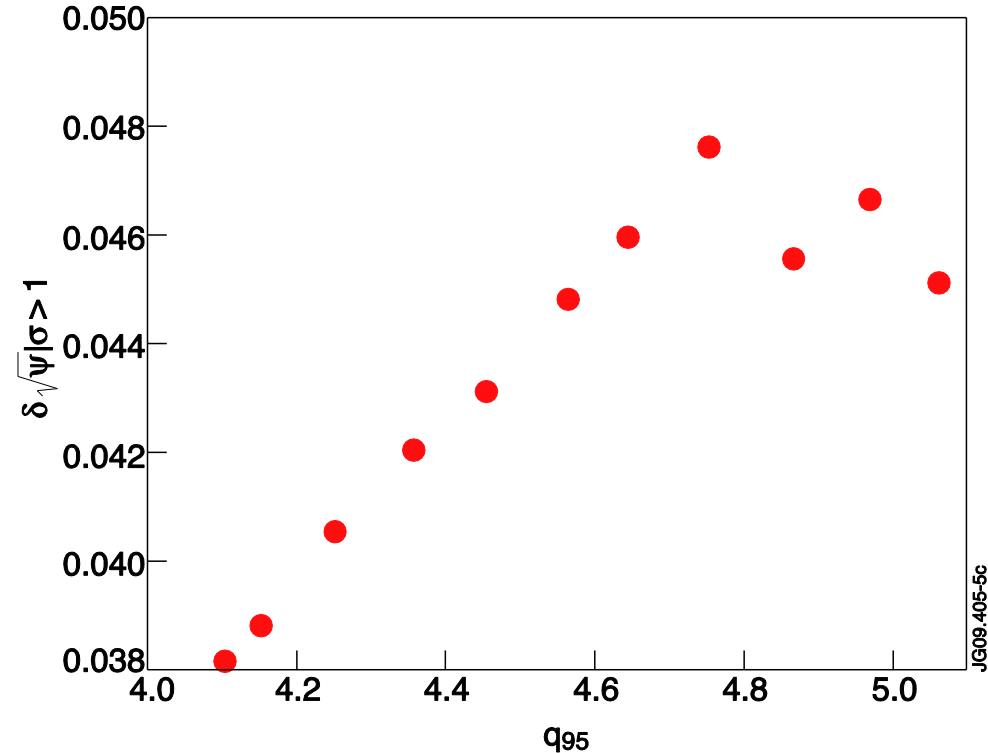
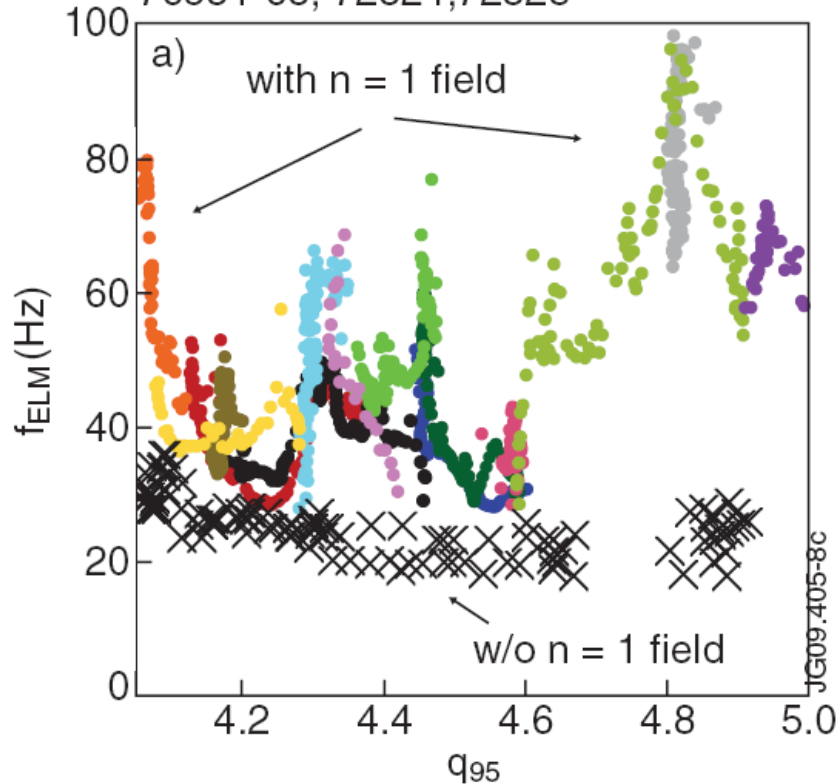


Active ELM control (frequency/size) observed in a wide q_{95} window, but no ELM suppression

Y Liang, et al, PRL, 98, 265004 (2007)
 Y Liang, et al, PPCF, 49, B581 (2007)
 Y Liang et al, JNM, 390–391, 733–739 (2009)

JET Pulse No's:
76951-65; 72524,72528

Y. Liang et al., PRL 105, 065001 (2010)



- Multiple resonances in f_{ELM} vs q_{95} have been observed with $n = 1$ and 2 fields
- Possible explanation in terms of ideal peeling mode model [C G Gimblett et al., PRL, **96**, 035006-1-4(2006), J Pearson, Y Liang et al, Nucl. Fusion 52 (2012) 074011].

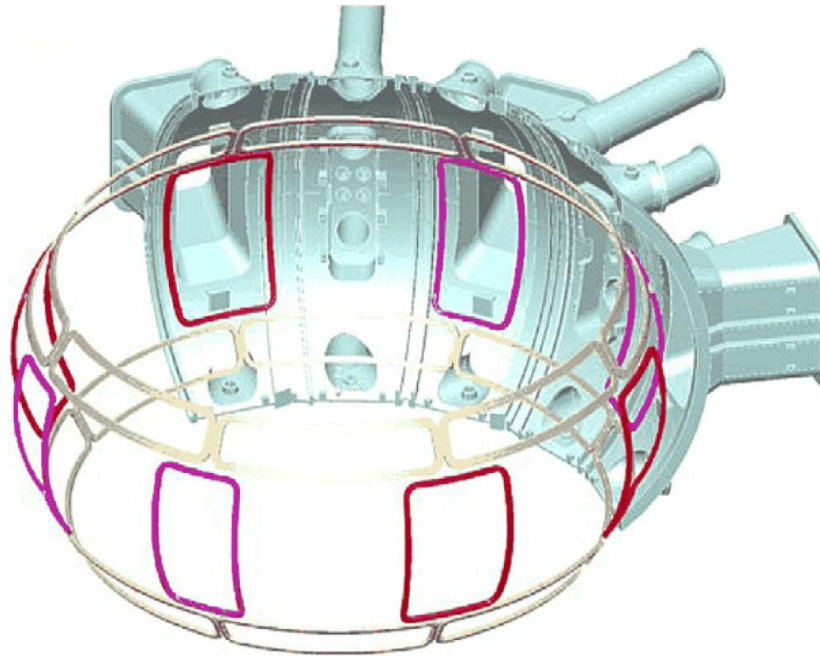
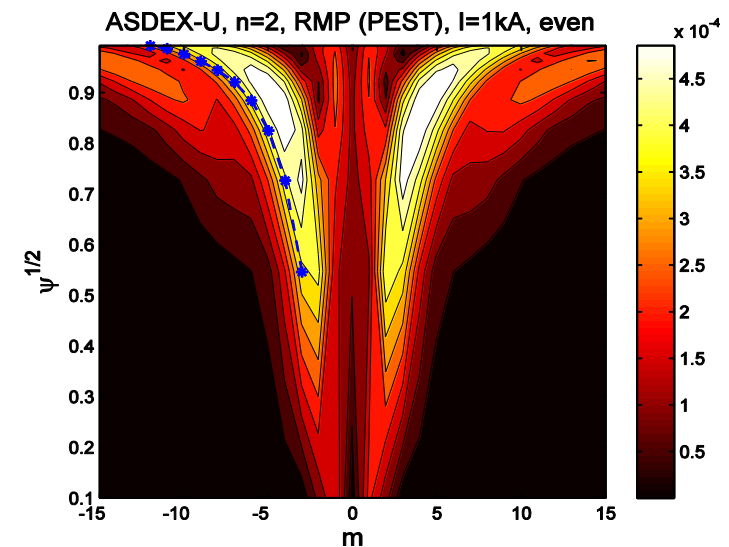
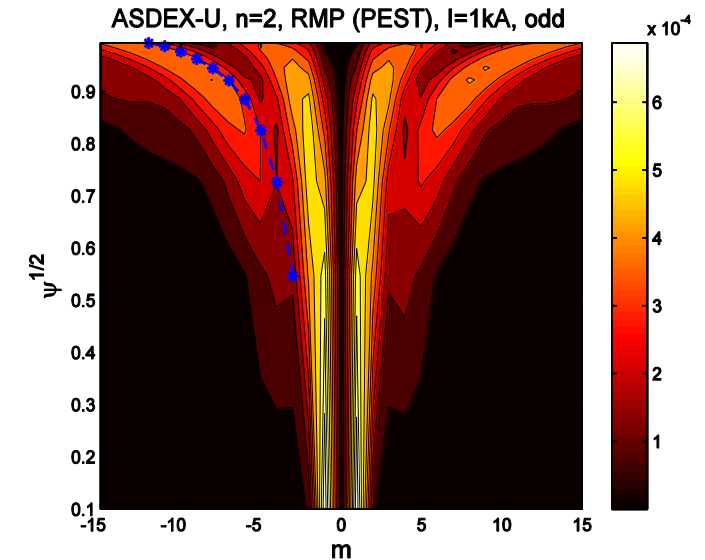


Fig. 1. 3D view of active in-vessel coils.

W Suttrop, Fusion Engineering and Design 84 (2009) 290

In 2011: Two rows $\times 4$ toroidally distributed coils ($n = 2$).
Single DC supply (all coils in series / anti-series).

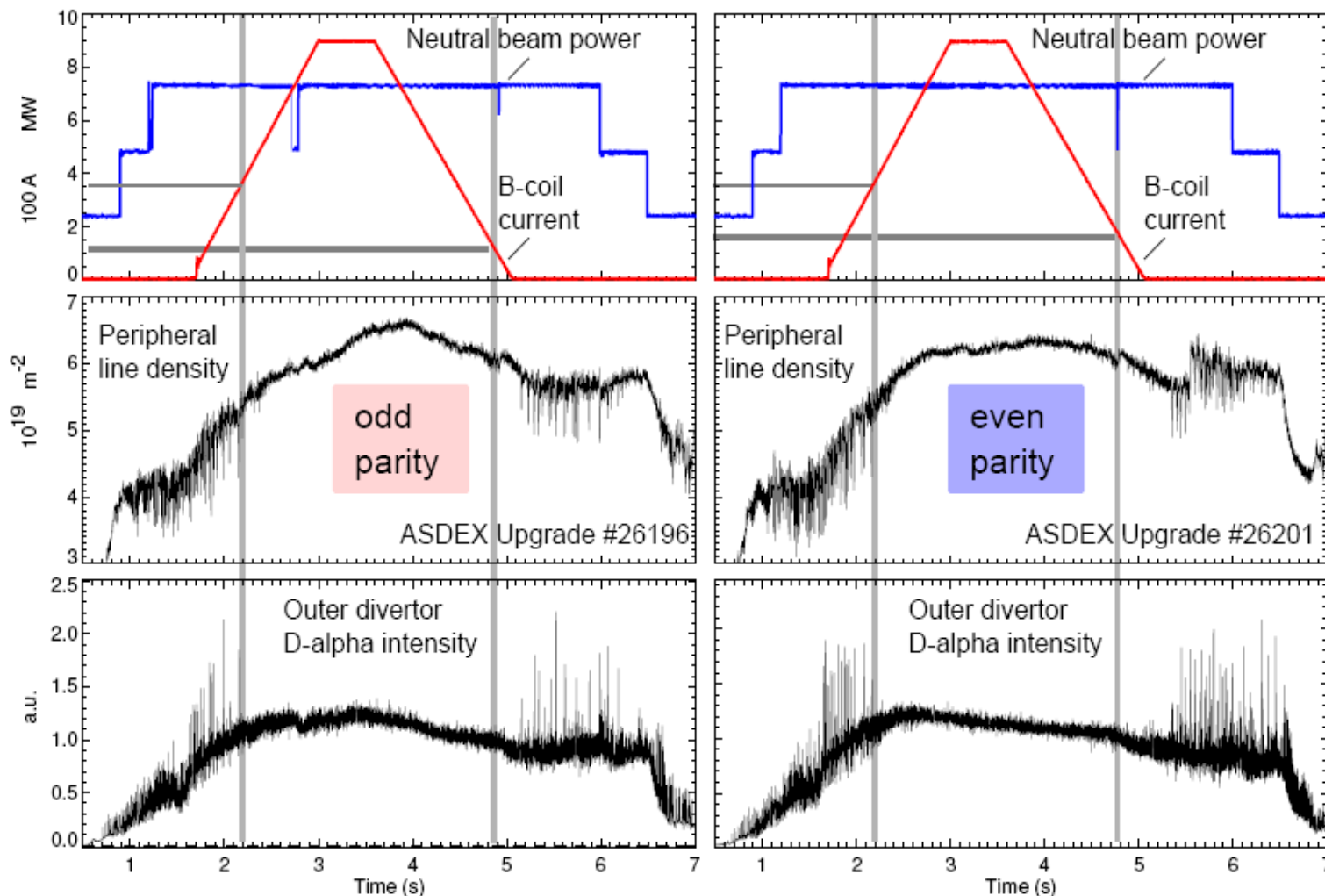


Identical coil current threshold for odd and even parity



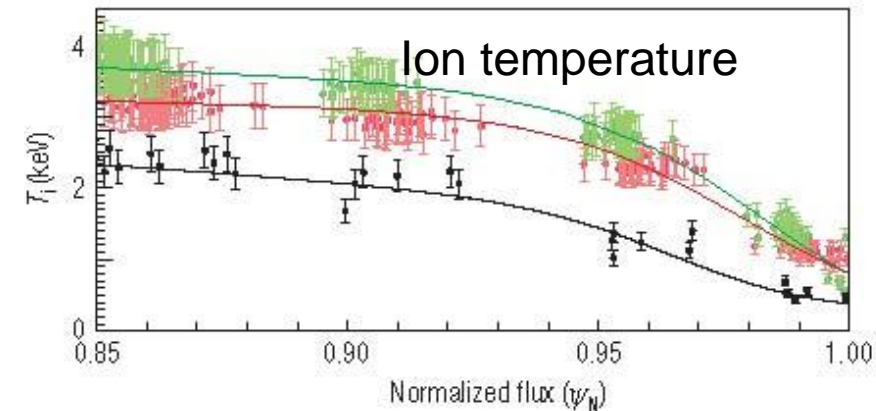
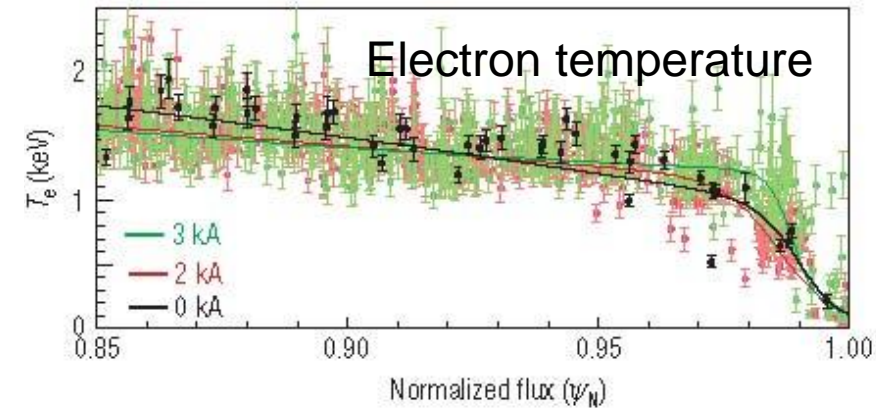
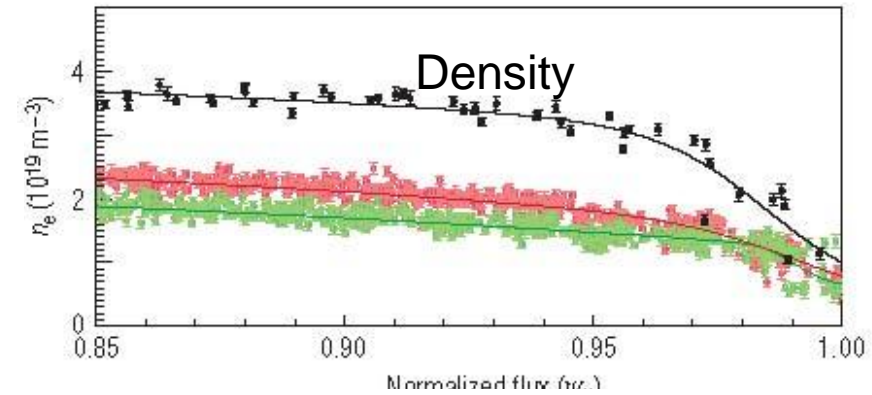
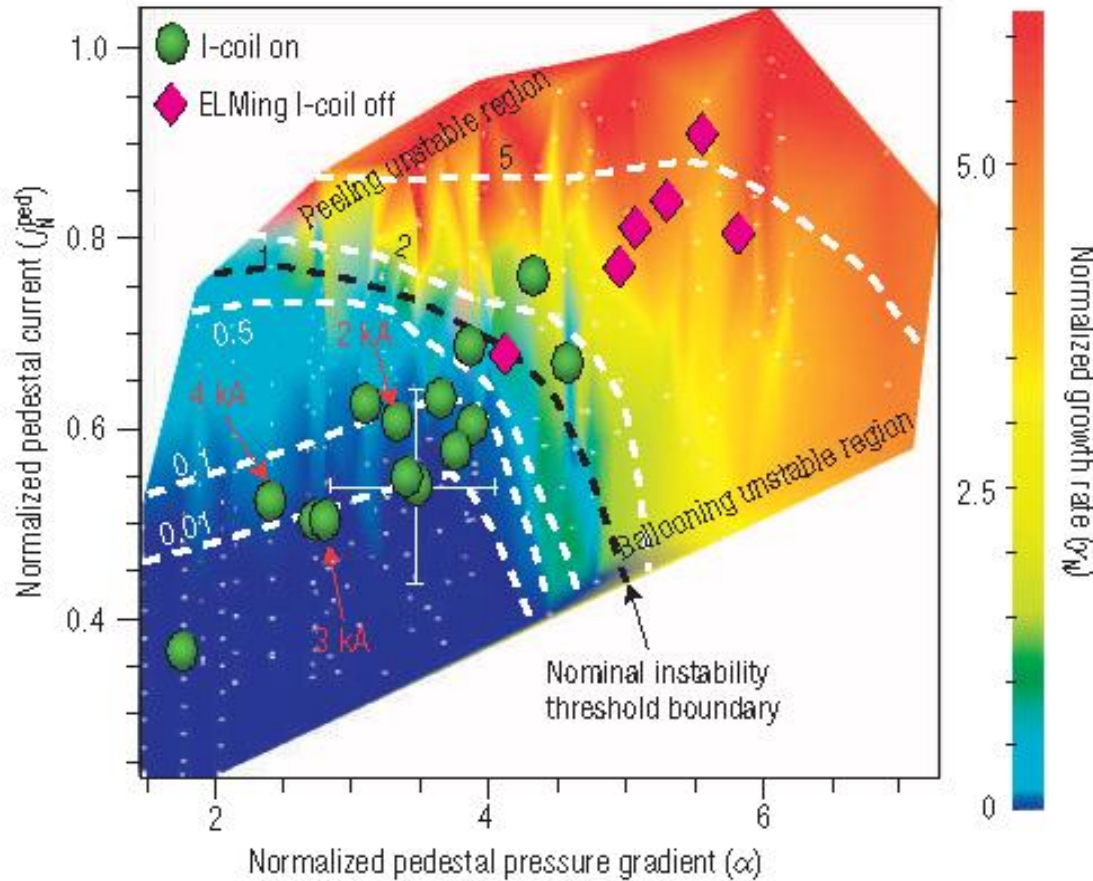
W Suttrop, SFP workshop (2011)

Hysteresis: $I_{\text{coil}} = 350$ A to and ≈ 150 A from ELM-mitigated state



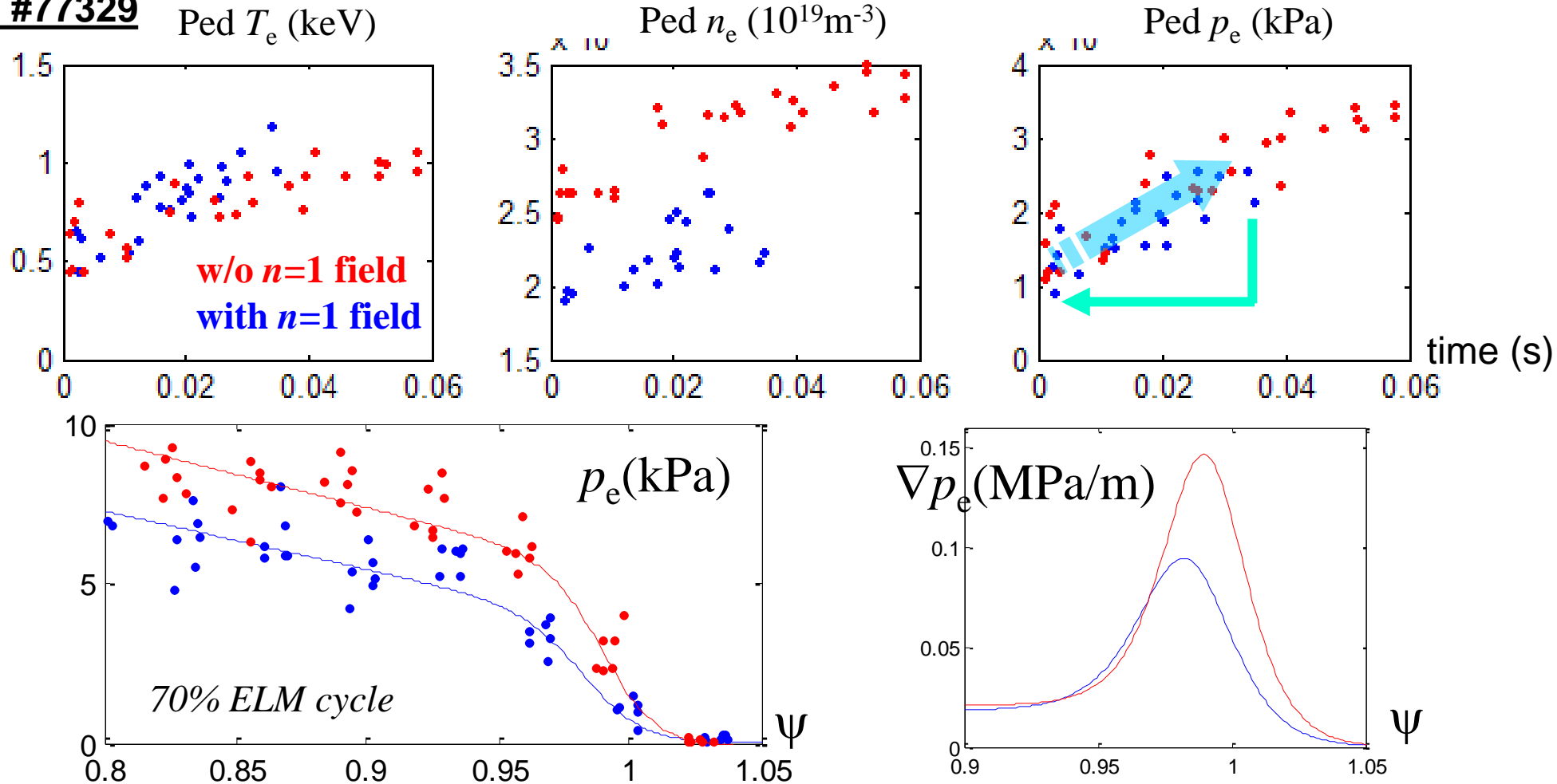
Dominant mechanism of ELM suppression

T. E. Evans, et al., *Nature physics*, Vol. 2, p419, June 2006



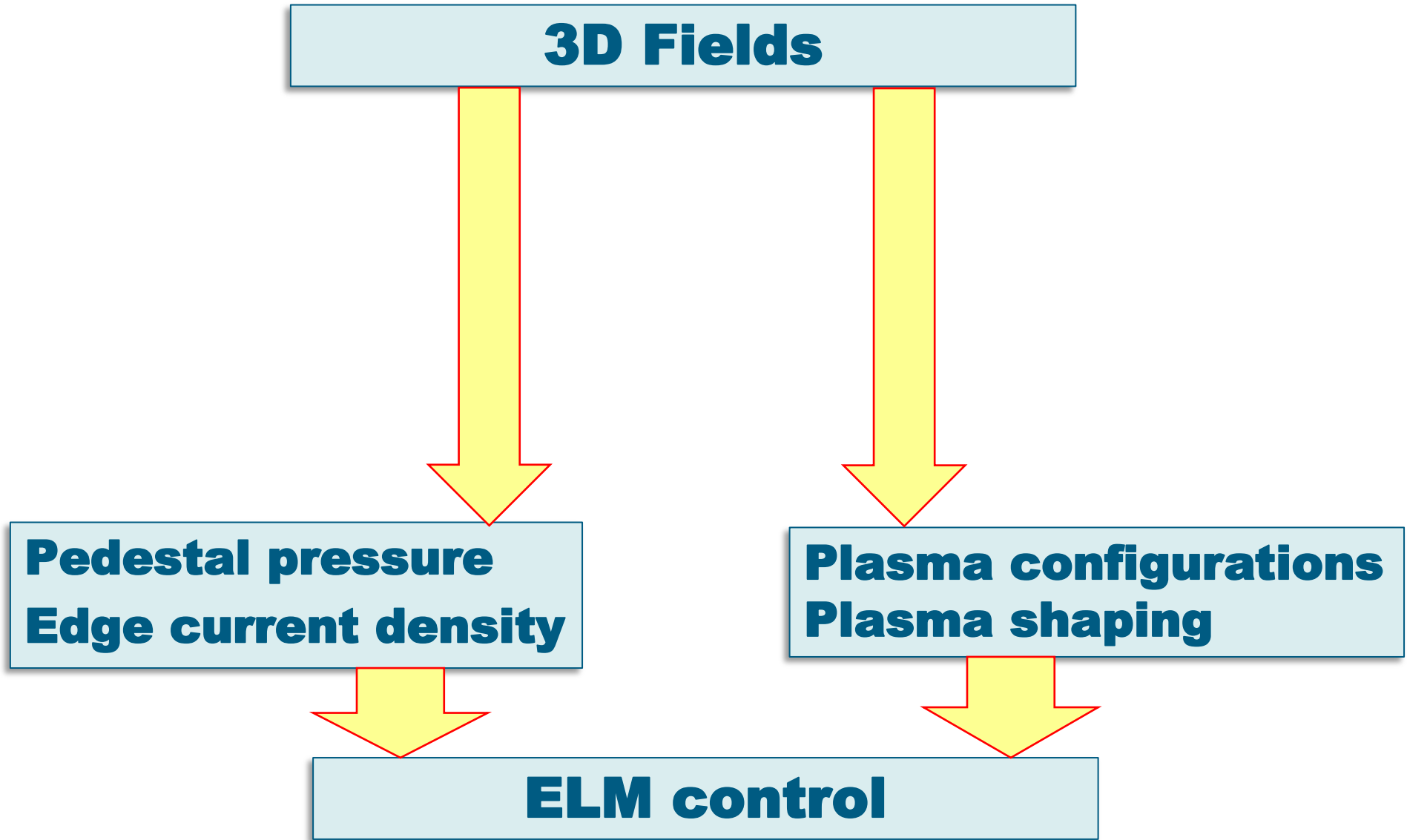
Reduction of edge pressure below instability threshold

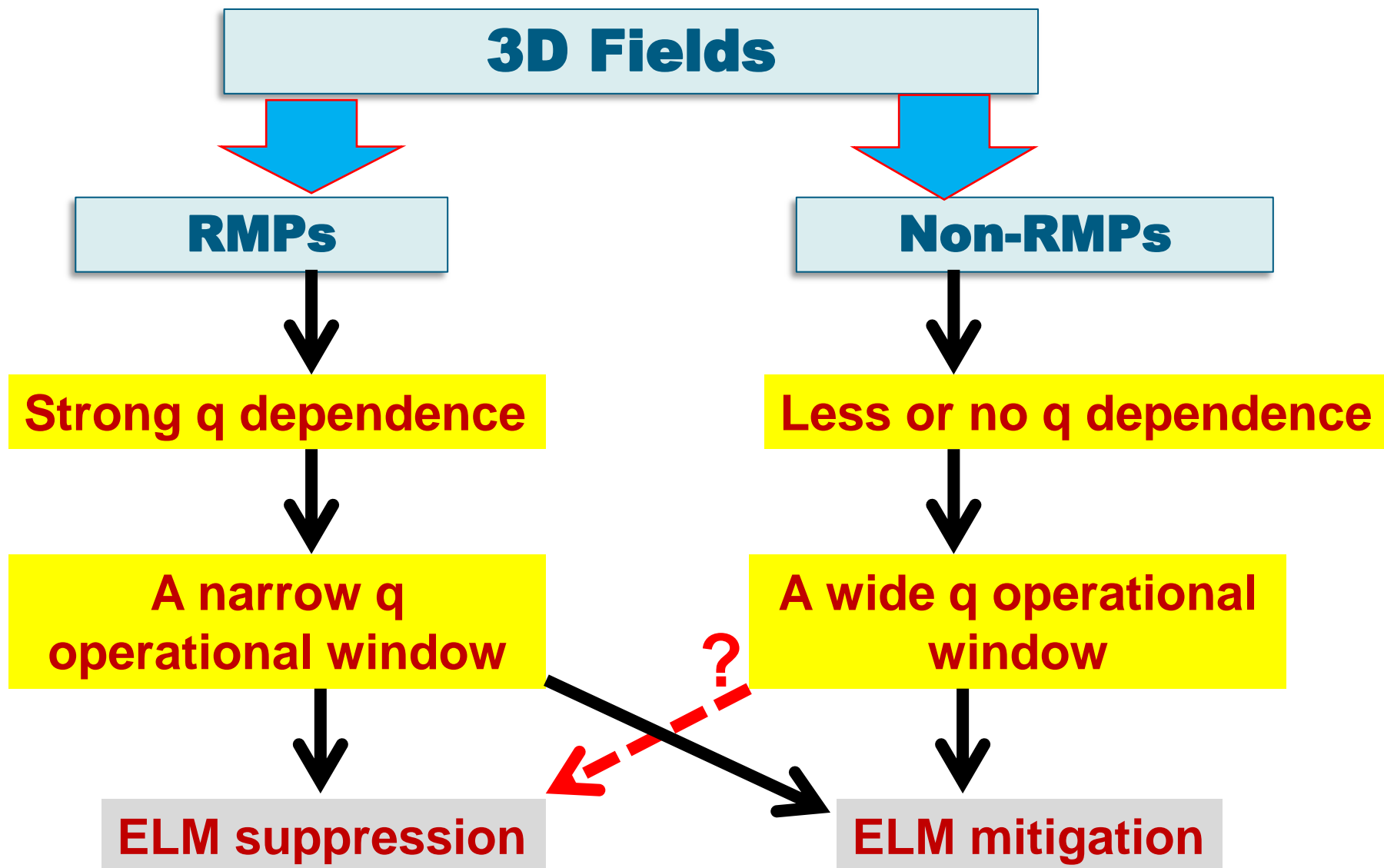
JET #77329

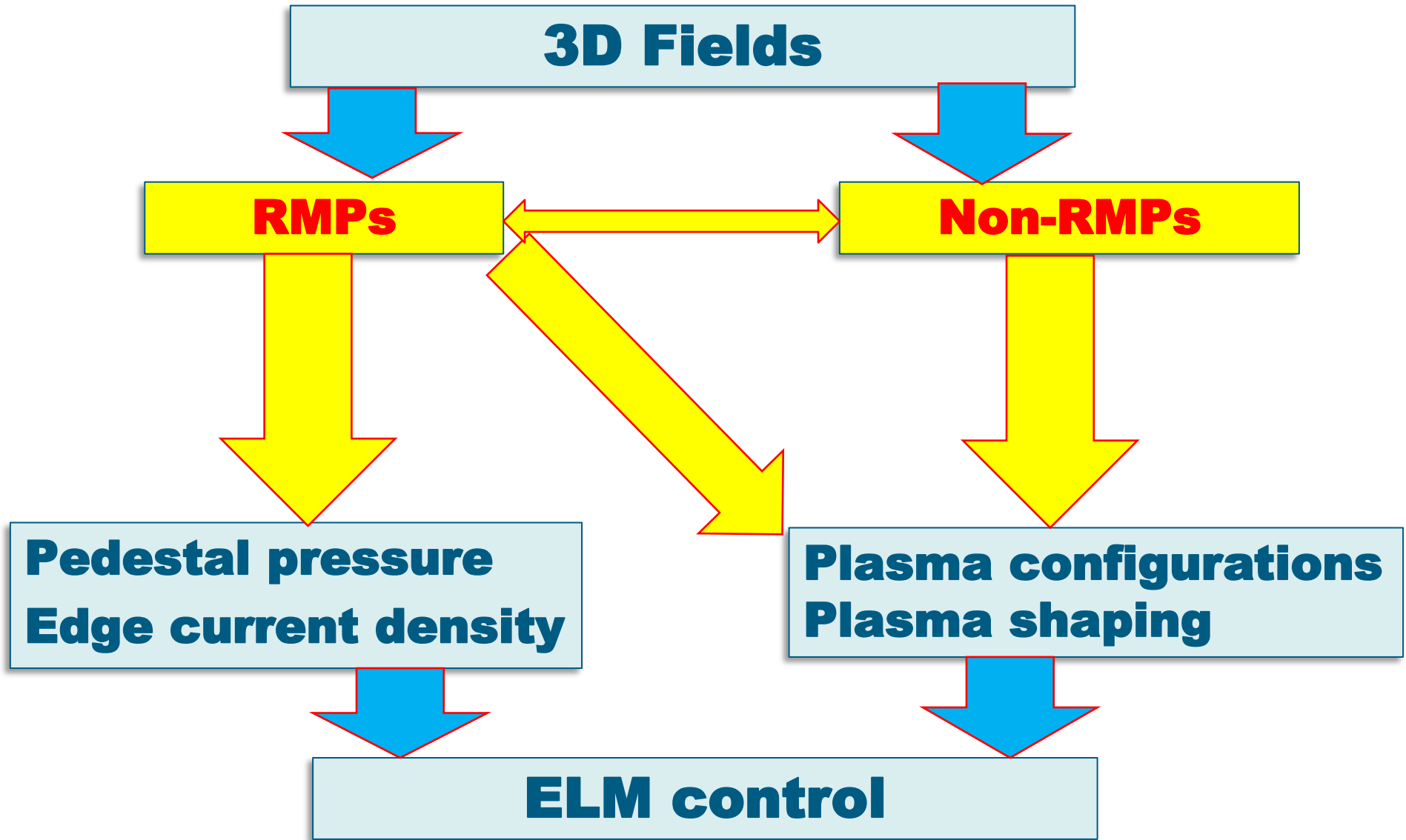


Y. Liang et al., Plasma and Fusion Research: 5, S2018 (2010)

- Pedestal pressure with $n = 1$ field applied recovers at same rate, but the ELM crash occurs earlier at lower $p_{e,ped}$.
- Pedestal n_e is reduced by $\sim 20\%$ while the edge T_e is increased. ∇p_e is $\sim 20\%$ smaller.









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Magnetic Field 'Frozen in' to the Fluid

In a perfectly conducting fluid

(**Ideal MHD**) $E + v \times B = 0$.

- The magnetic flux through each surface moving with the fluid is constant and consequently that the magnetic flux can be thought of as "**frozen-in**" to the fluid and moving with it.

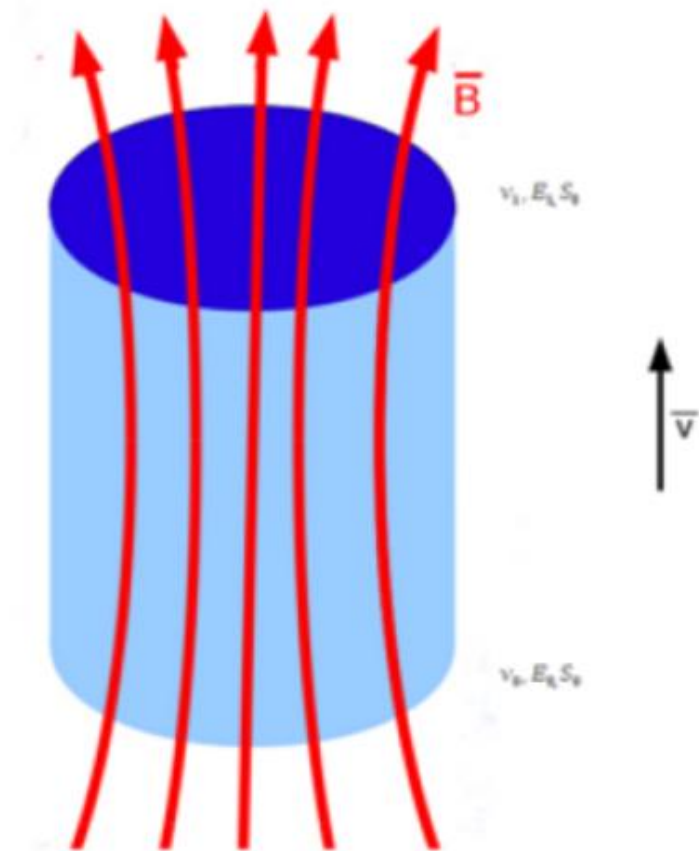
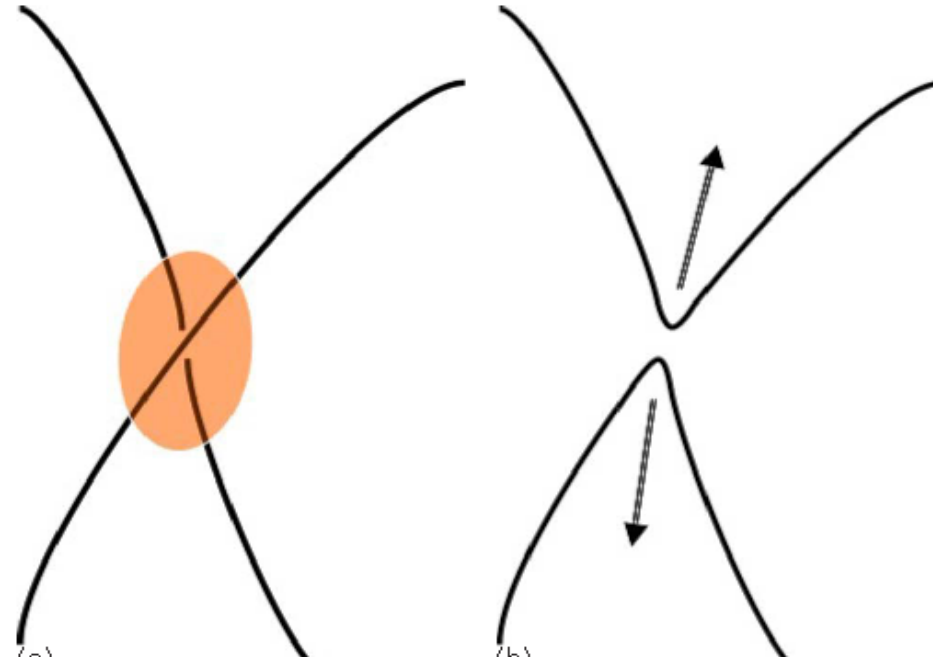


Figure 1.1: sketch to illustrate a fluxtube

Non-ideal (**Resistive**) modes: $\mathbf{E} + \mathbf{v} \times \mathbf{B} = \eta \mathbf{j}$



- When the field lines are reconnected, the topology of magnetic configuration changes and $\mathbf{j} \times \mathbf{B}$ forces result in the conversion of magnetic energy to kinetic energy.

Resonant Magnetic Perturbation (RMP)

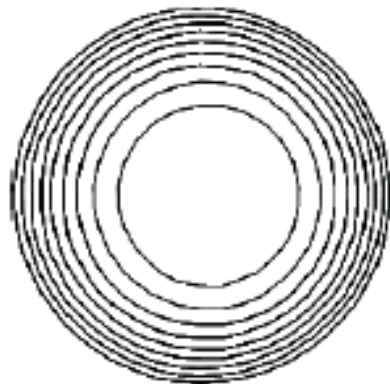
Magnetic field perturbation: $\vec{b} = \nabla\varphi \times \nabla\tilde{\psi}$
 $\tilde{\psi}(\varphi, \vartheta, r) = \tilde{\psi}_0(r) \cdot e^{i(n\varphi - m\vartheta)}$

$q(r) = \frac{m}{n}$ – inside the plasma

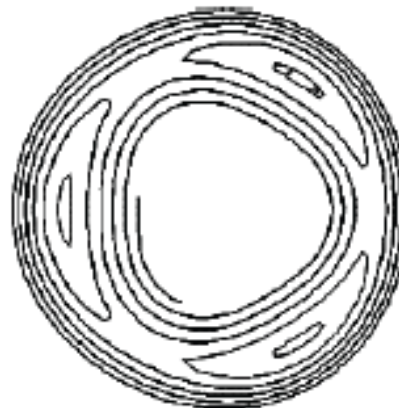
$q(r) = \frac{m}{n}$ – outside the plasma

Resistive Responses

Ideal Responses

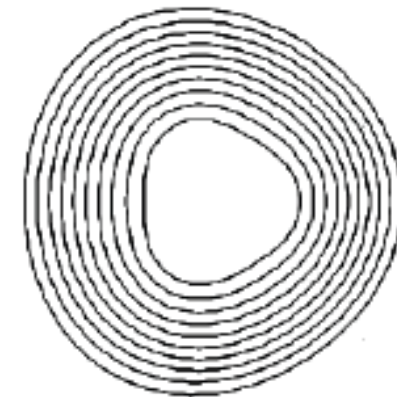


Unperturbed Magnetic Surfaces



Internal (Tearing) Modes

or

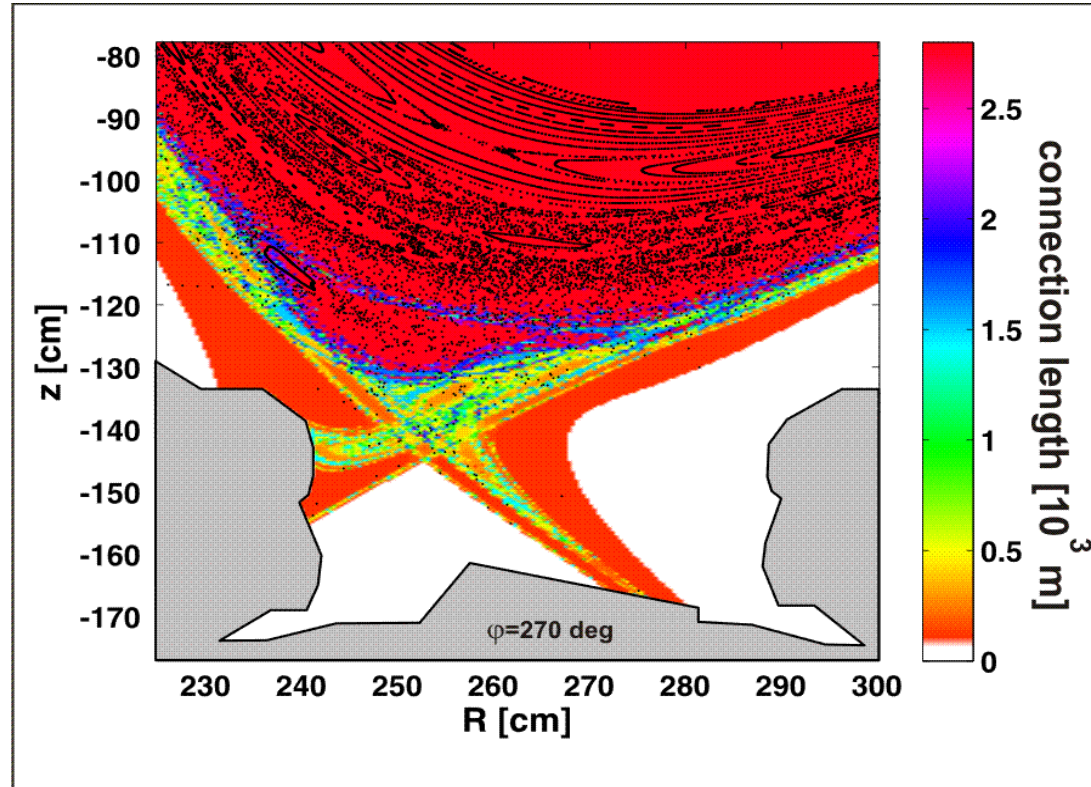


External Kink Modes



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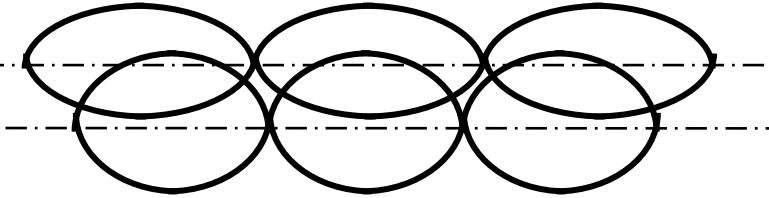
Equilibrium Magnetic Field at Plasma Edge



Edge Ergodisation with a magnetic perturbation

$$q = m/n$$

$$q = (m+1)/n$$



Chirikov parameter

$$\sigma_{m,m+1} = \frac{W_{n,m} + W_{n,m+1}}{2\delta_{m,m+1}}$$

larger than 1

- ❑ Splitting of strike point
- ❑ Spin-up plasma rotation to co-current direction

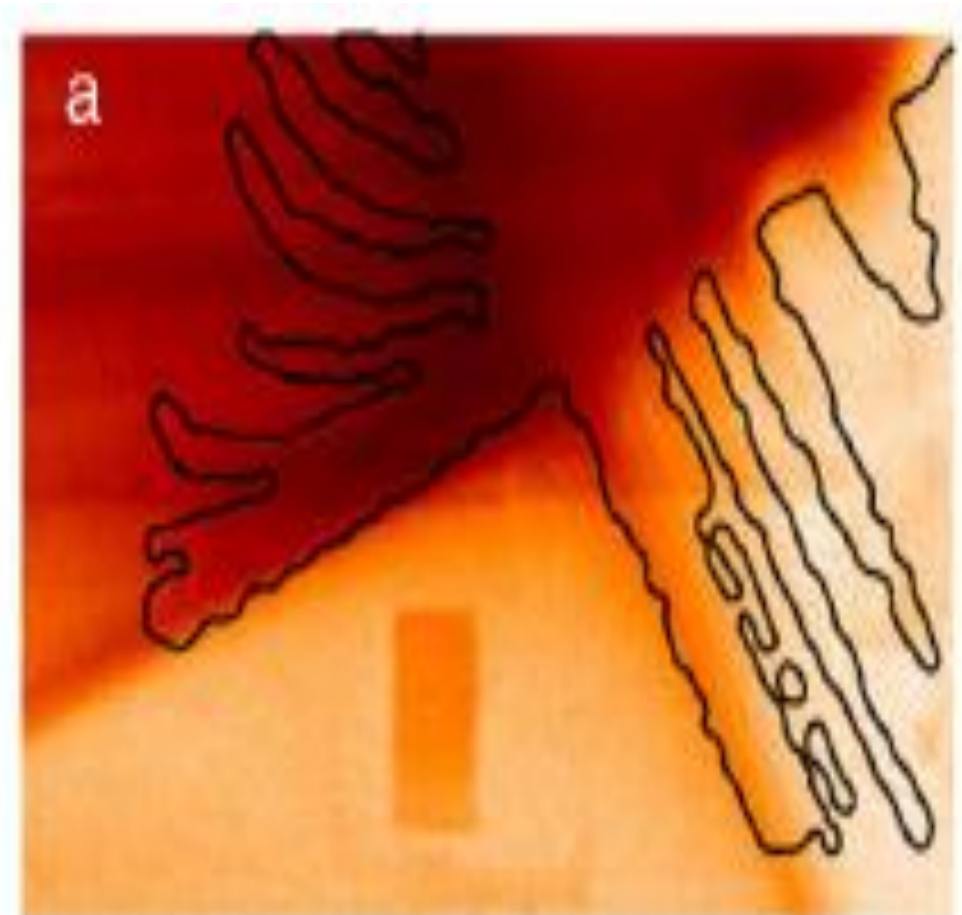
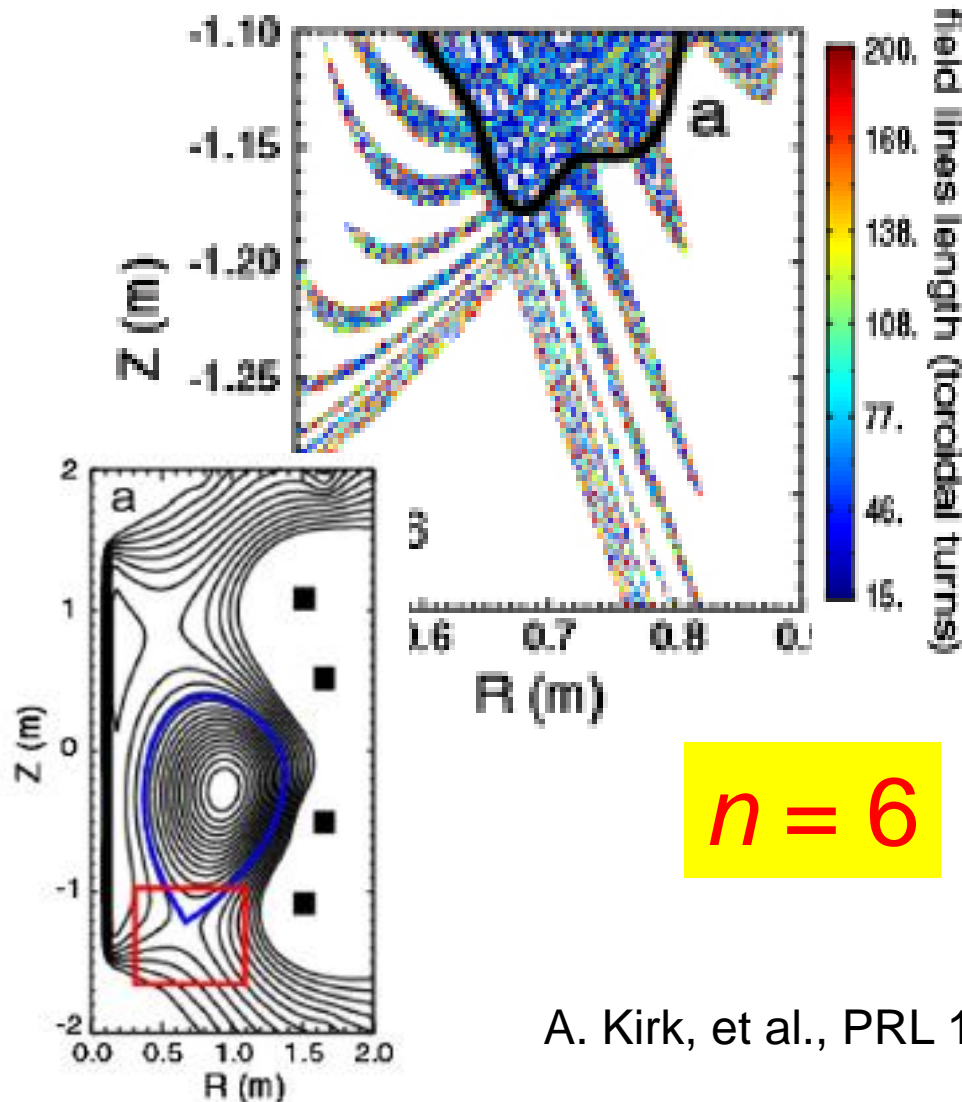


O. Schmitz, PPCF (2008)

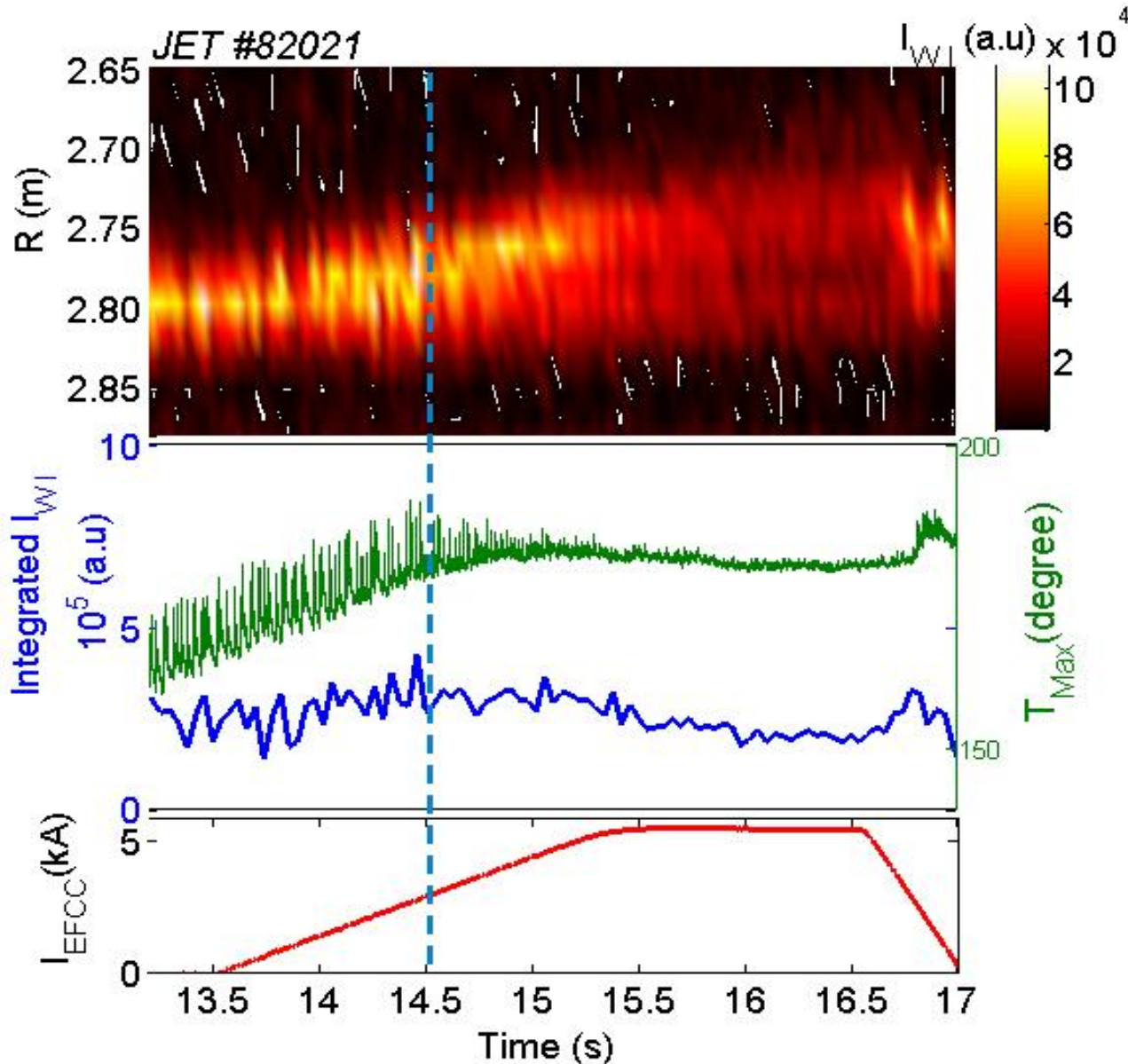
I. Joseph JNM, 2007

Splitting of the inner strike-point has been observed during ELM suppression with an $n = 3$ field on DIII-D.

Observation of Lobes near the X Point in RMP Experiments on MAST



A. Kirk, et al., PRL 108, 255003 (2012)



High collisionality

With $n = 2$ field, splitting of the outer strike point and reduction of the erosion of the outer tungsten divertor have been observed during the mitigation of the large type-I ELMs.

Y. Liang, et al., IAEA, 2012

TEXTOR

Ohmic plasmas

6/2 DED

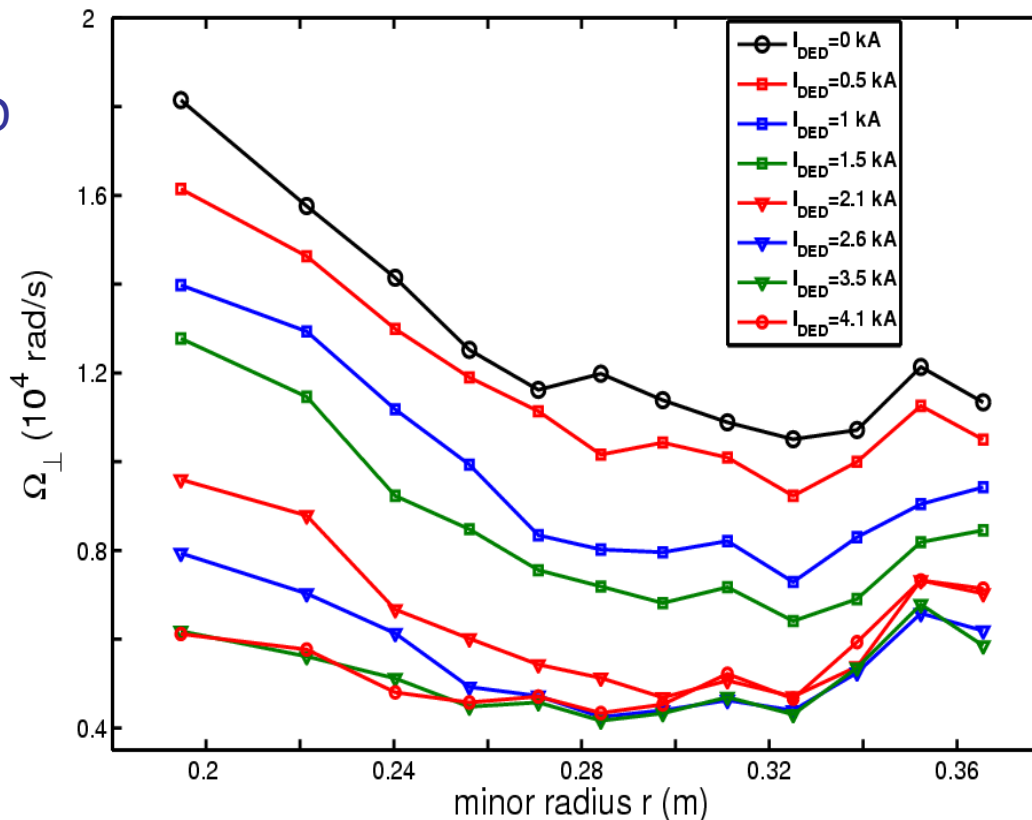
EDD

Increase I_{DED}

IDD

T Zhang, Y Liang
et al., SFP 2011

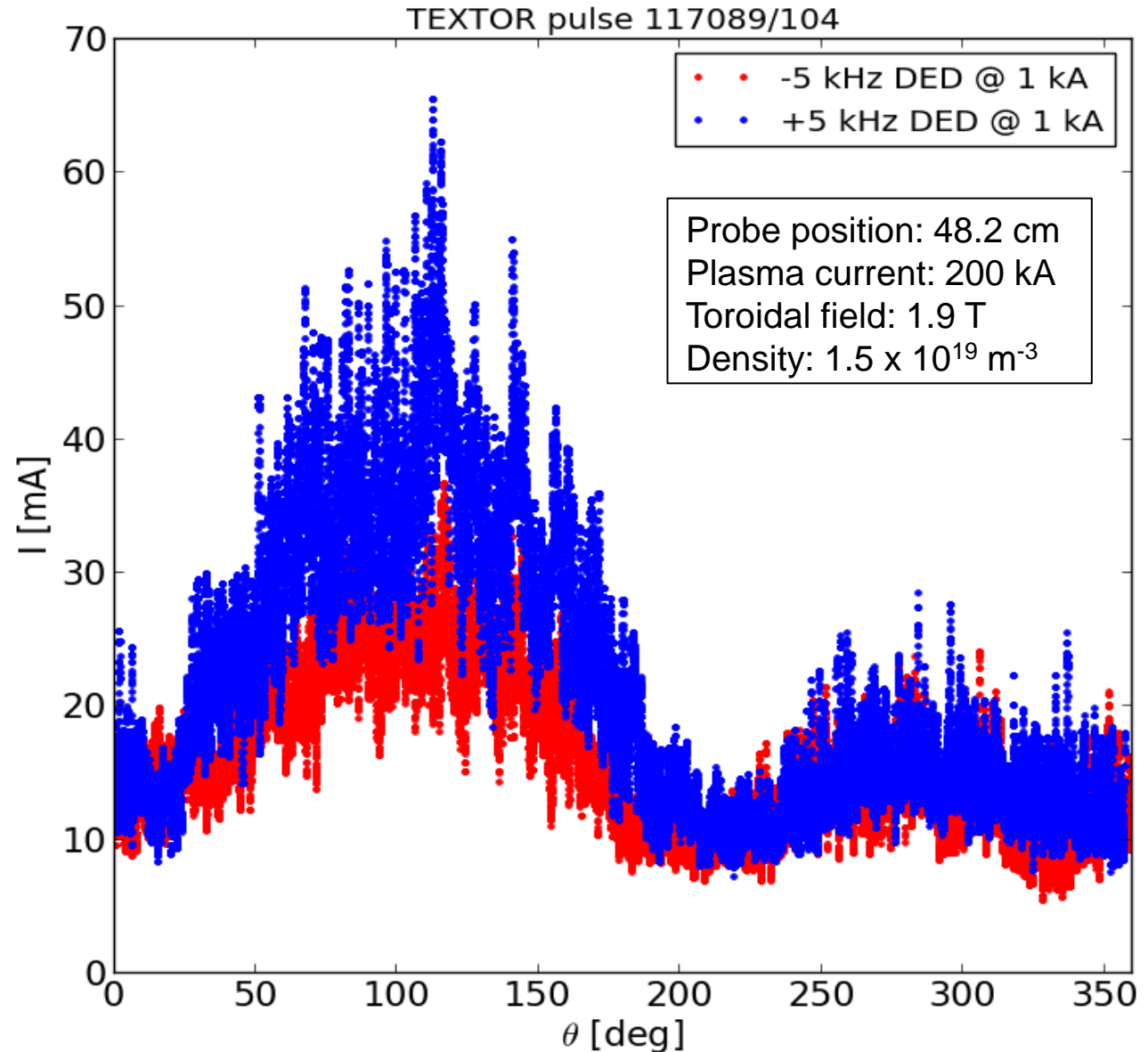
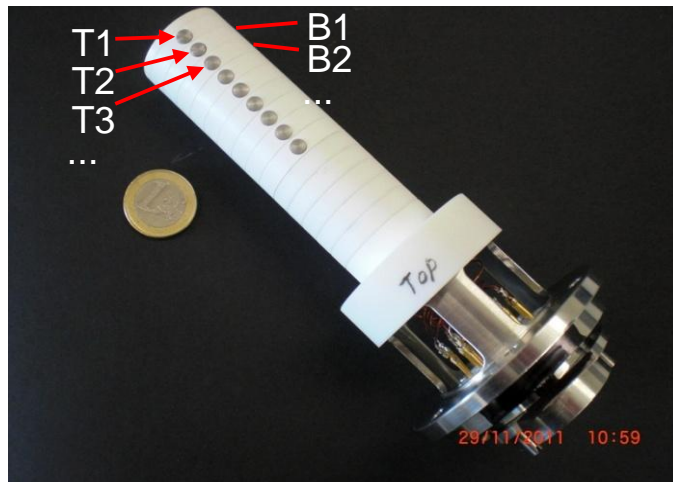
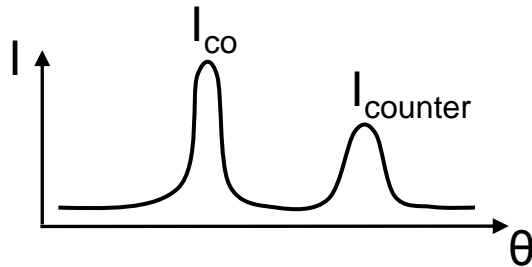
Ω_{\perp} profile evolution with DED current



✓ Spin up plasma rotation in IDD direction with $m/n=6/2$ DED

Properties of FMDP:

- Negative biasing to measure ion saturation currents
- Measurement of radial profile
- Measure θ dependence of ion saturation currents and fast ion losses by rotating the probe



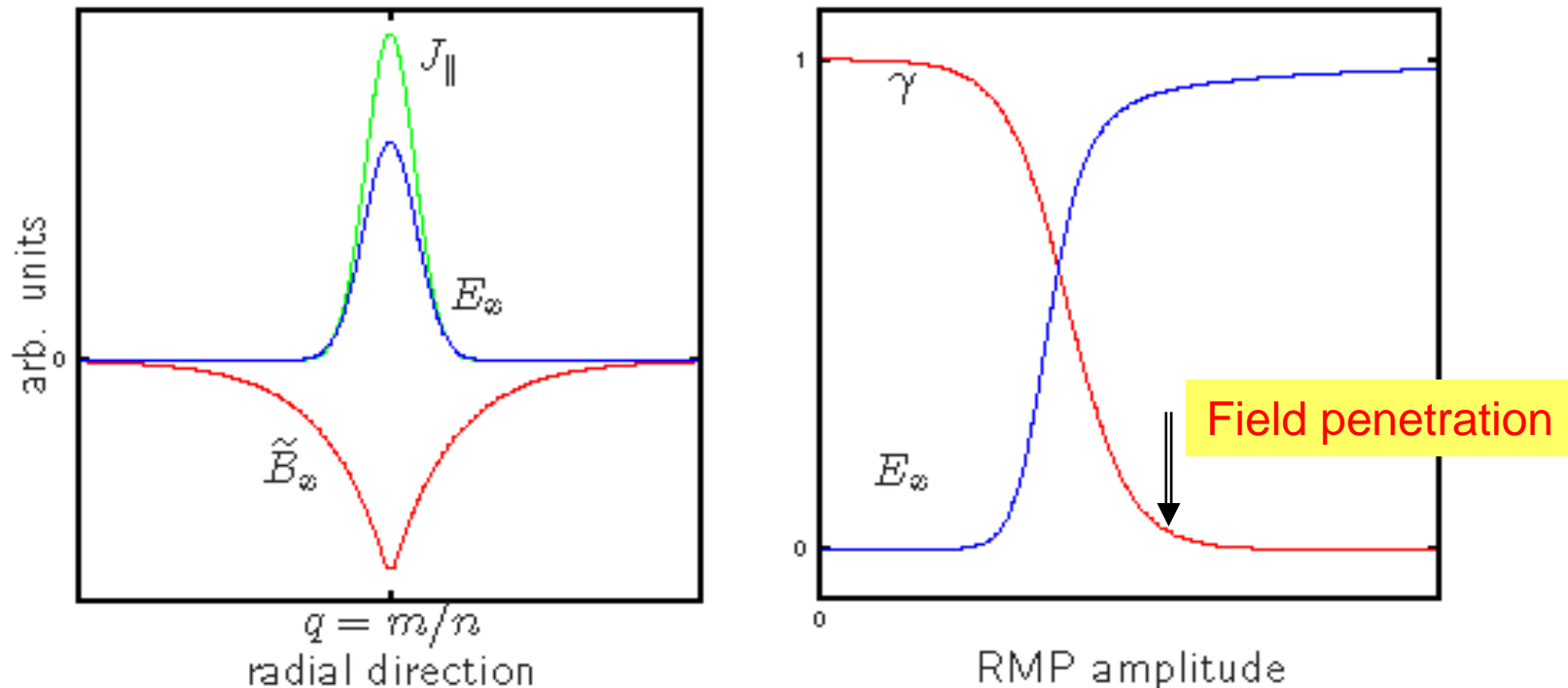


- Introduction to ELMs and 3D fields on tokamak
- Summary of ELM control with 3D fields
 - ELM suppression
 - ELM mitigation
- **What are the possible physical mechanisms of ELM control with 3D fields?**
 - Resistive Plasma Responses
 - Field Penetration / Mode Excitation
 - Edge Ergodisation
 - **Ideal Plasma Responses**
 - **Rotation Screening Effect**
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What is the Rotational Screening Effect?

The rotational screening of RMPs results from the motion of the electron fluid across the field lines at the resonant surfaces.

$$V_{\perp,e} = V_{ExB} + V_e^*$$



D. Reiser and M. Z. Tokar, Phys. Plasmas **16**, 122303 2009

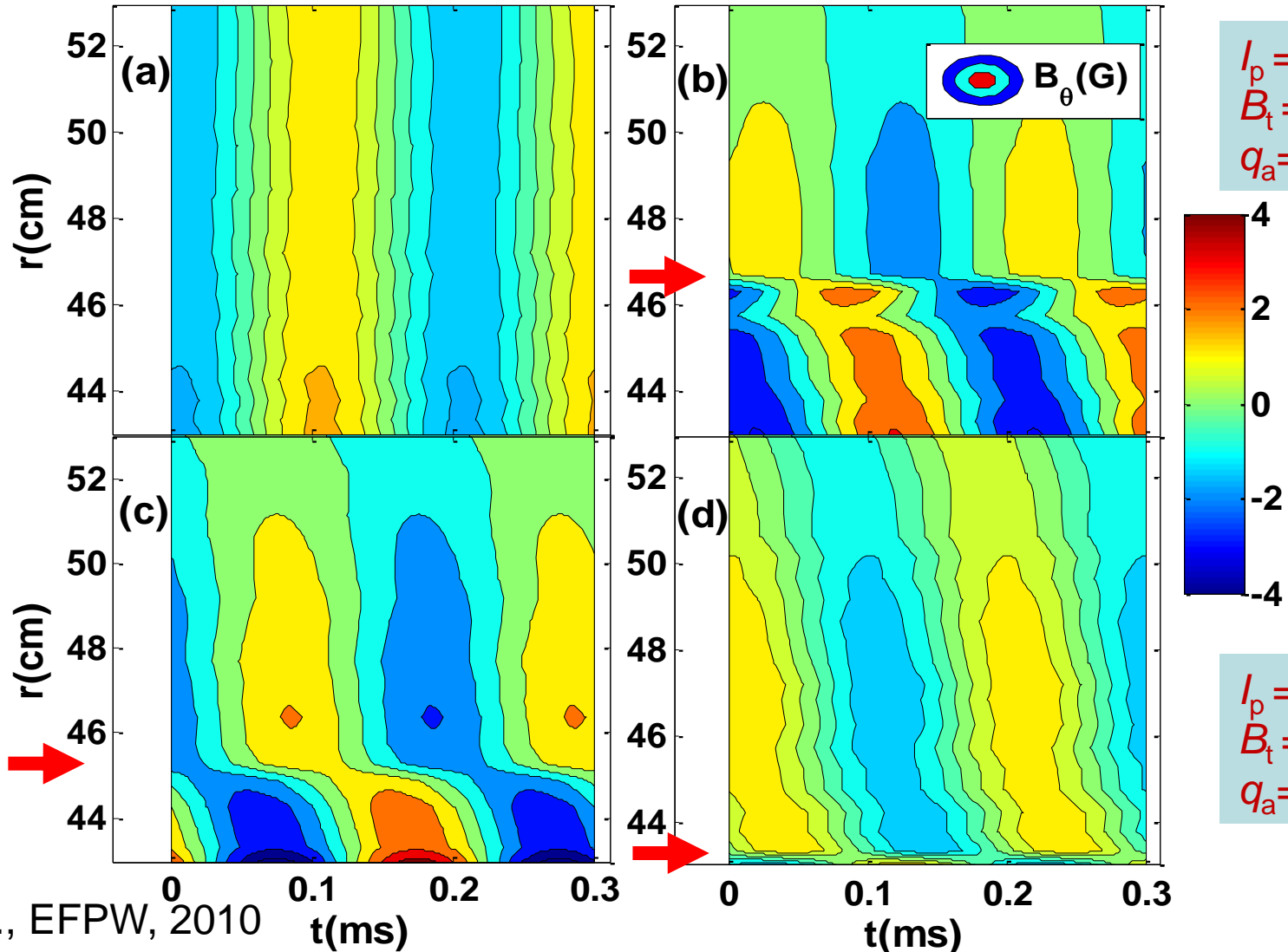
Edge safety factor (q_a) dependence of plasma response to $m/n=3/1$ DED field

$f_{\text{DED}}=5\text{kHz}$

Co-current direction

TEXTOR #113871/86/91

Vacuum



$I_p = 300\text{kA};$
 $B_t = 1.6\text{T};$
 $q_a = 3.4$

$I_p = 290\text{kA};$
 $B_t = 1.6\text{T};$
 $q_a = 3.5$

$I_p = 250\text{kA};$
 $B_t = 1.6\text{T};$
 $q_a = 3.9$

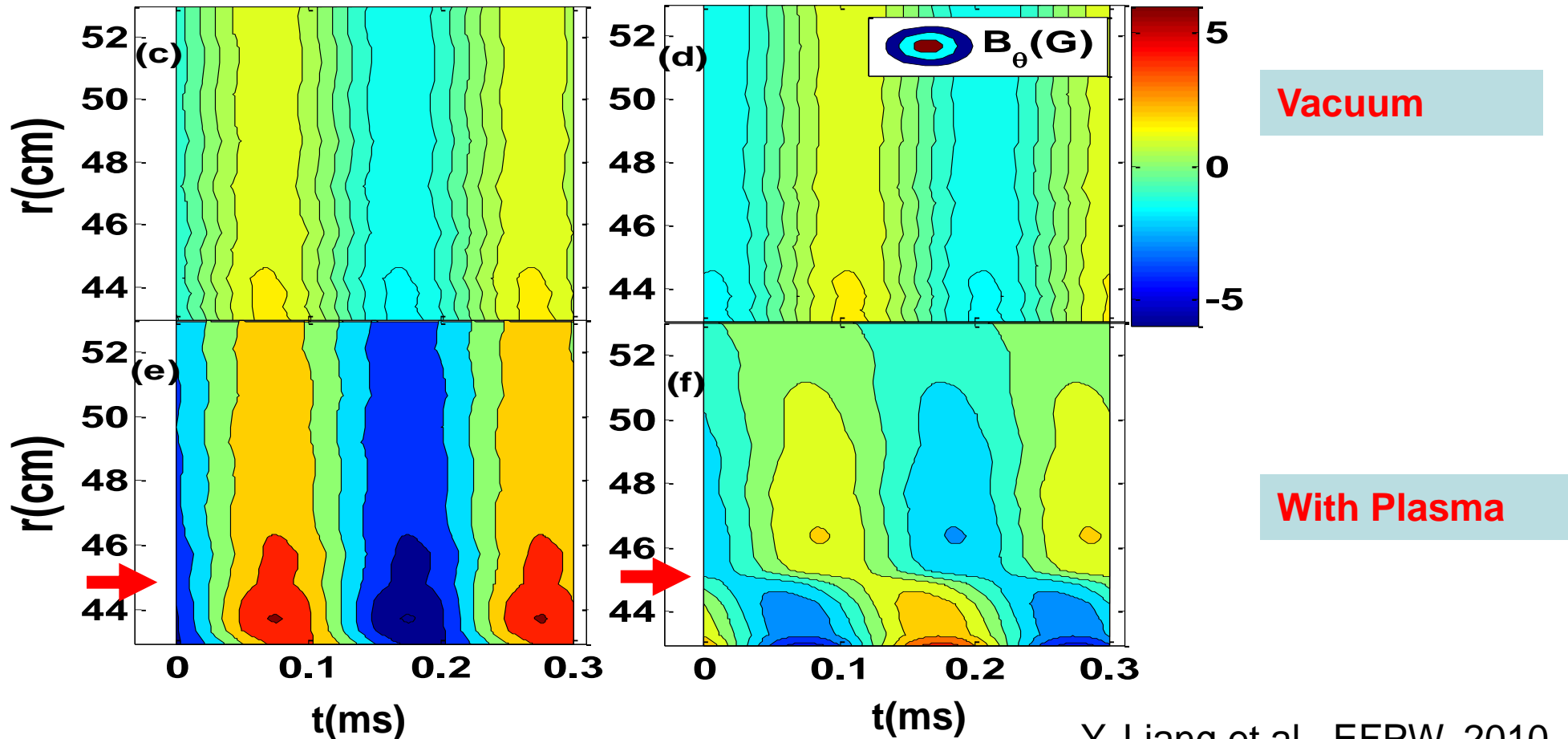
ctr-current direction

co-current direction

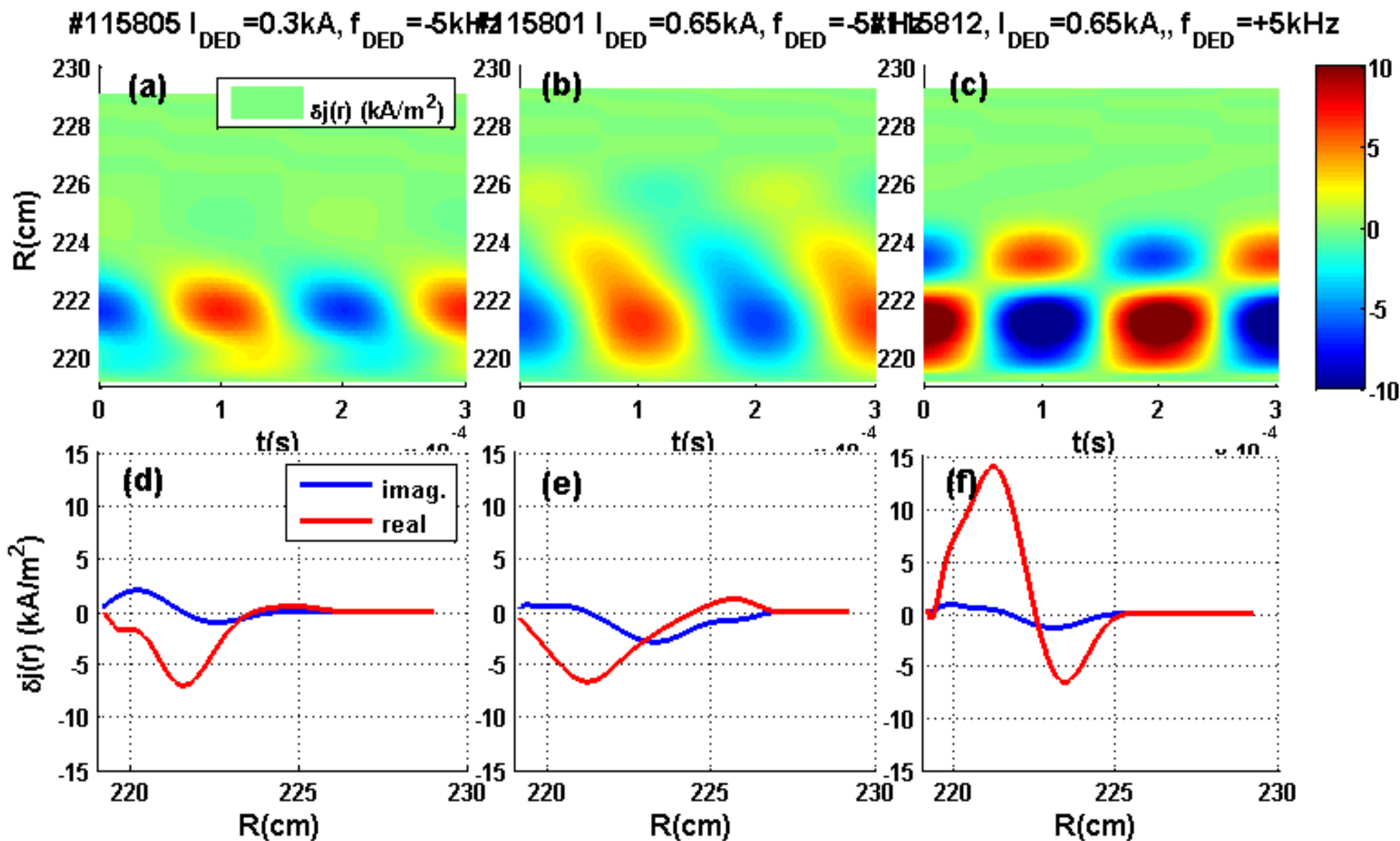
$I_p=250\text{kA}$; $B_t=1.6\text{T}$;
 $q_a=4.$; $n_e=1.0 \times 10^{19}\text{m}^{-3}$;

TEXTOR #113869/70/90/91

(c):113869; (d):113870; (e):113890; (f): 113891

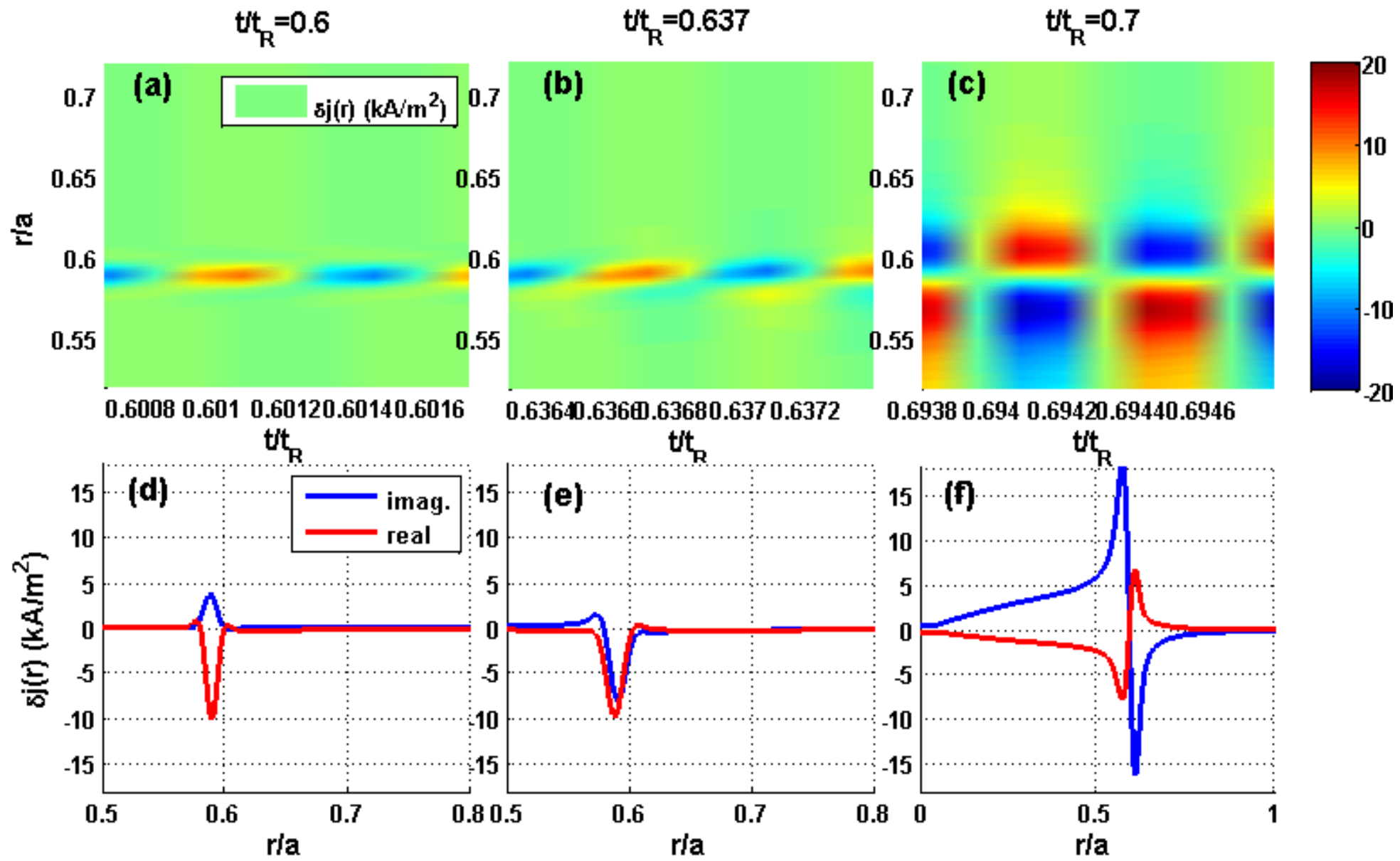


Y. Liang et al., EFPW, 2010

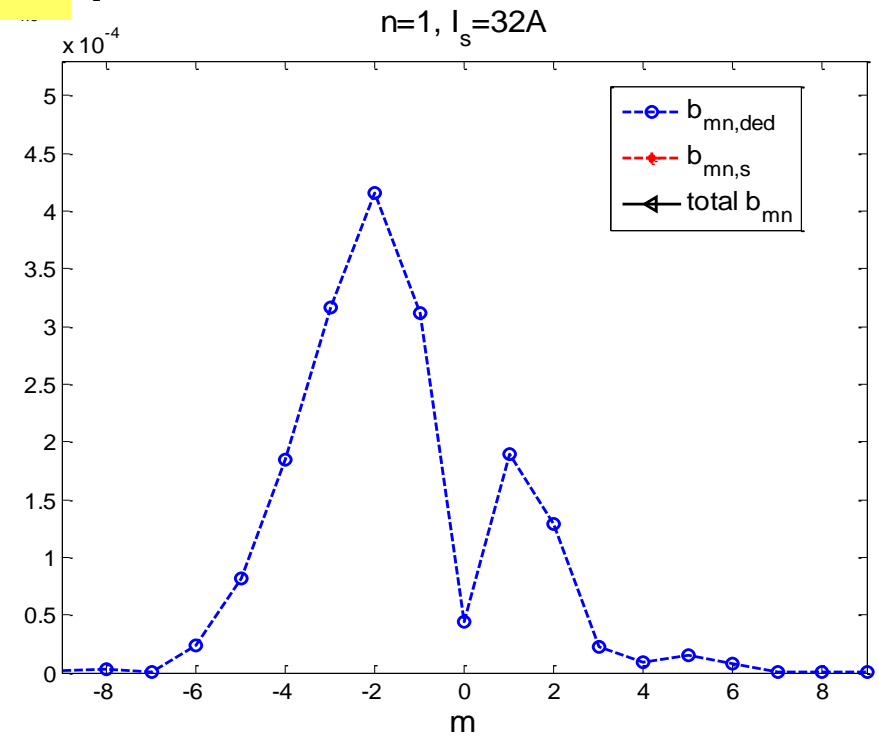
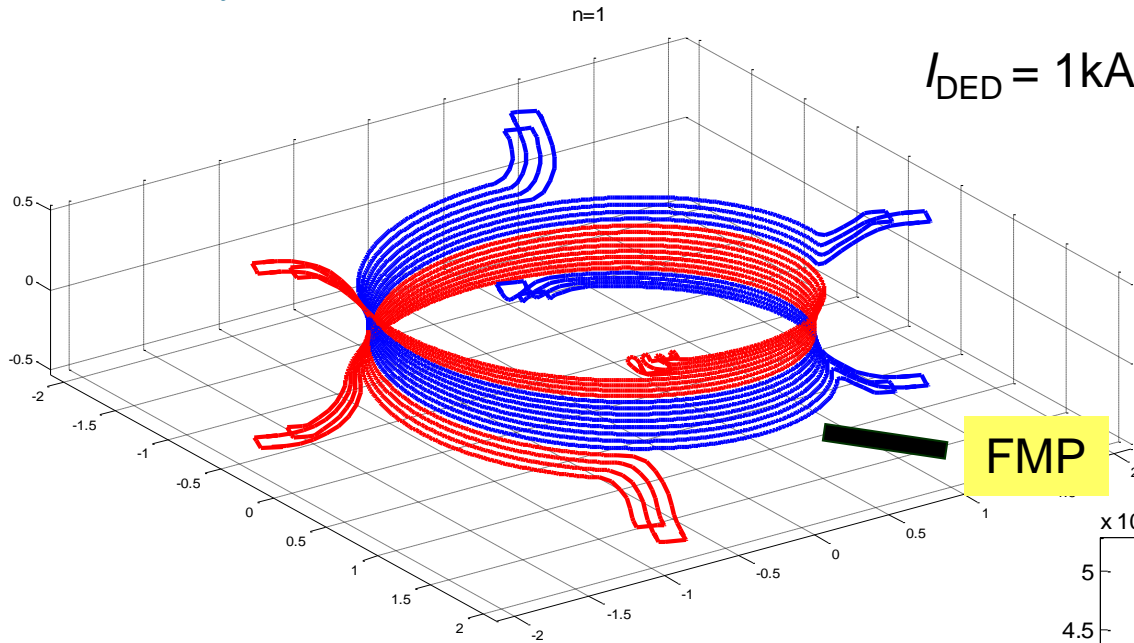


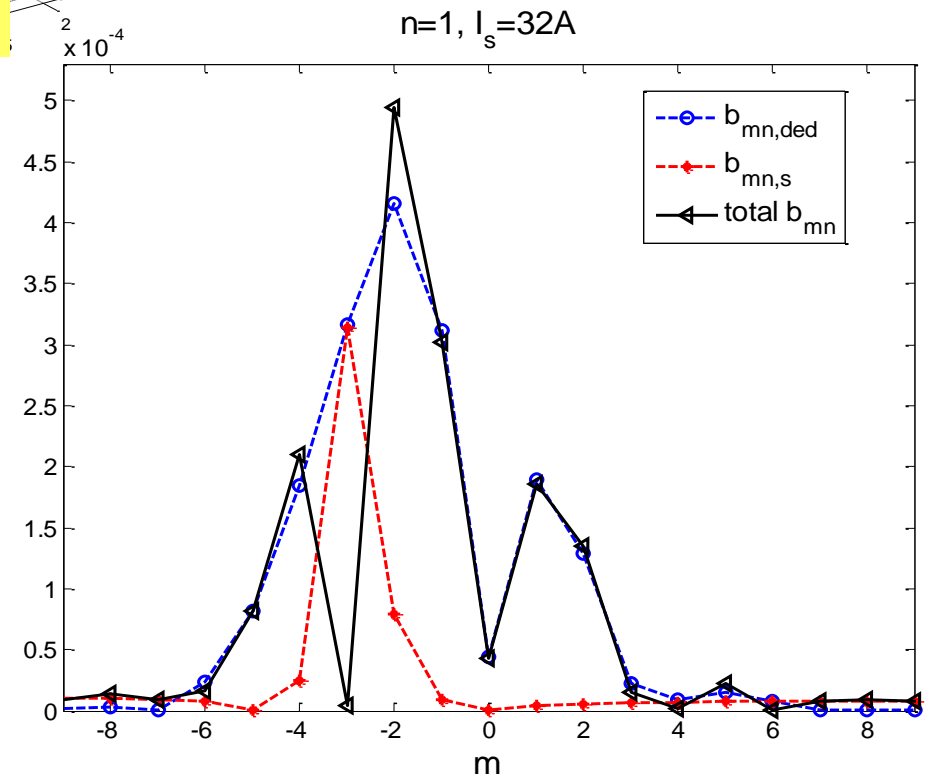
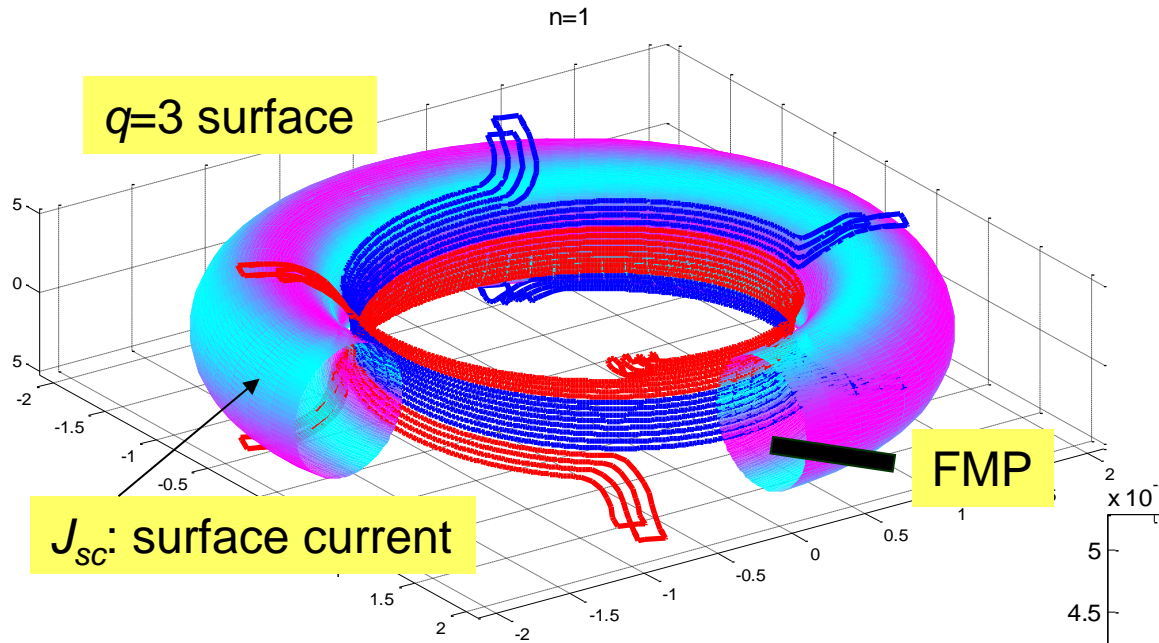
Y. Liang, et al., to be submitted to PRL, 2013

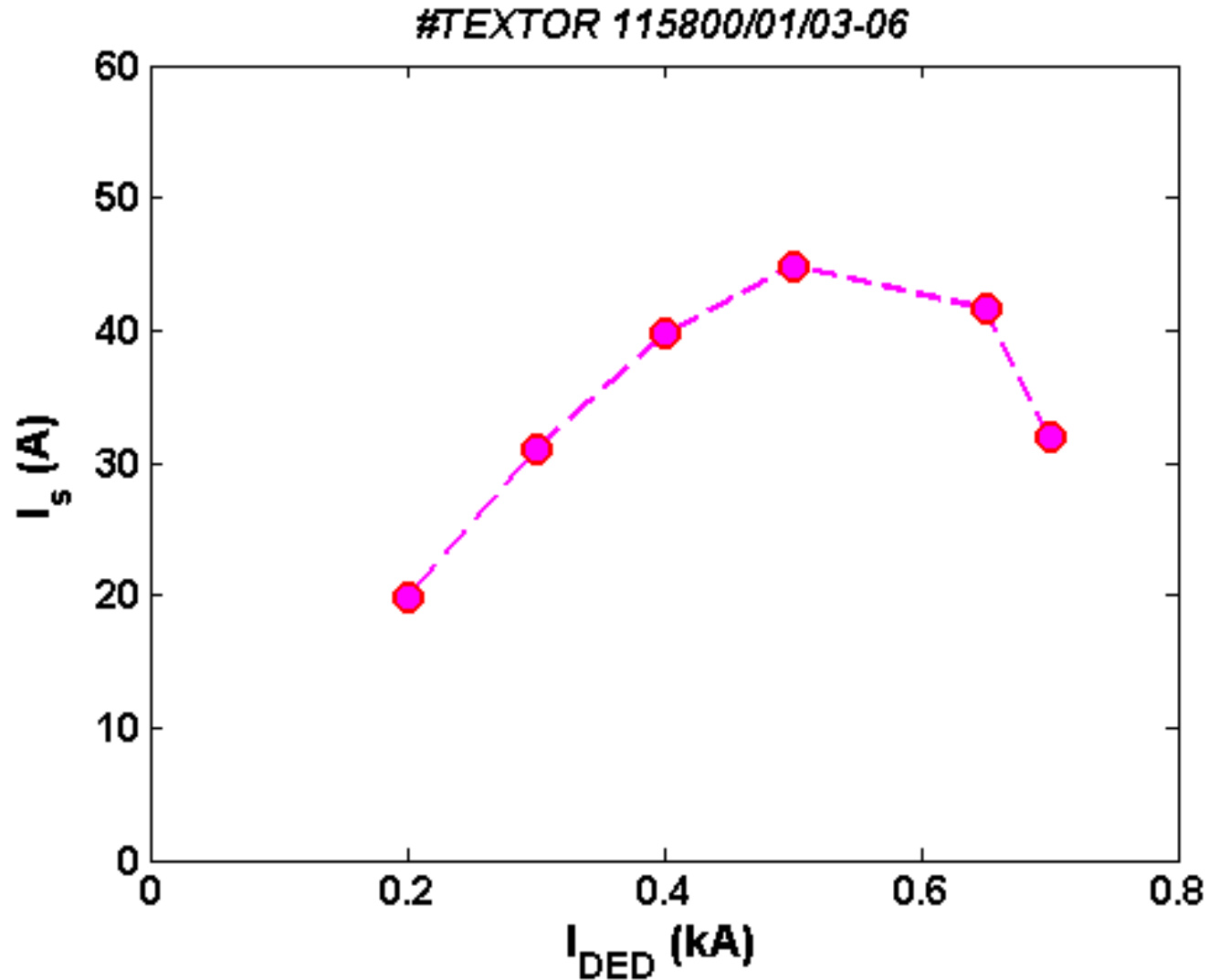
Modelling of Field Penetration Process



Y. Liang, et al., to be submitted to PRL, 2013

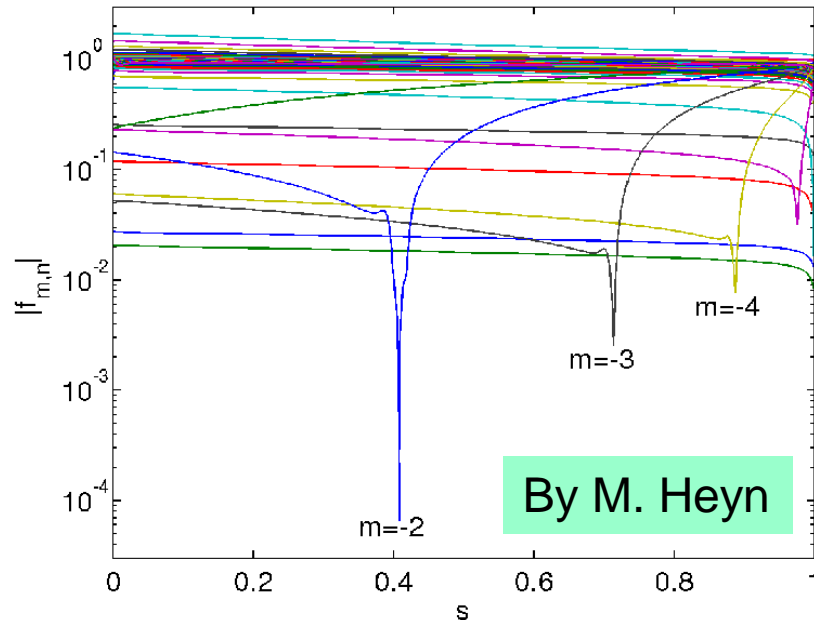




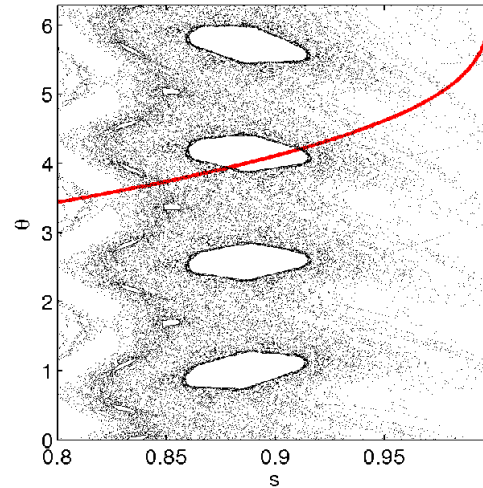


Y. Liang, et al., to be submitted to PRL, 2012

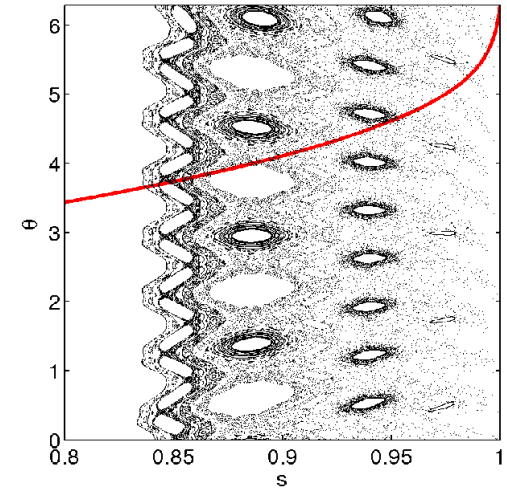
$$f_{mn} = B_{r,mn}^{(plas)} / B_{r,mn}^{(vac)}$$



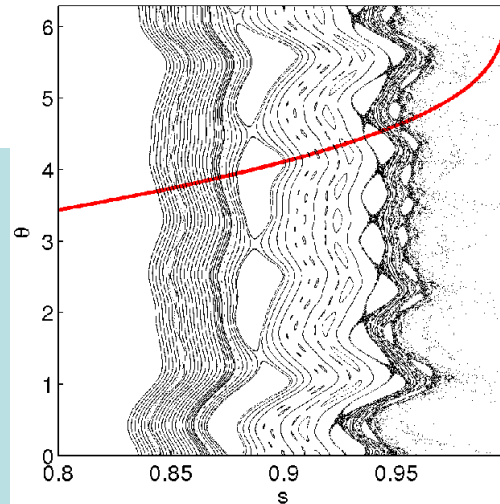
The resonant perturbation is shielded due to plasma rotation and the magnetic field topology in the plasma core is not affected by RMP's.



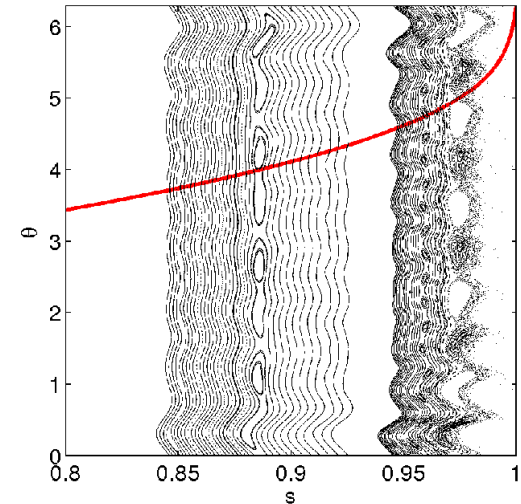
vacuum, $n = 1$



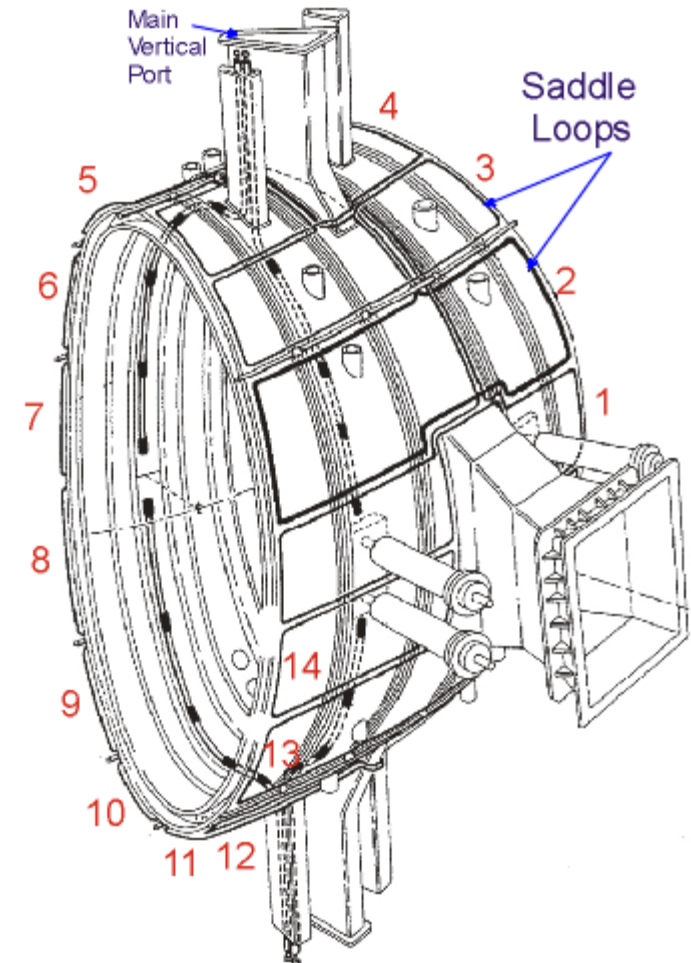
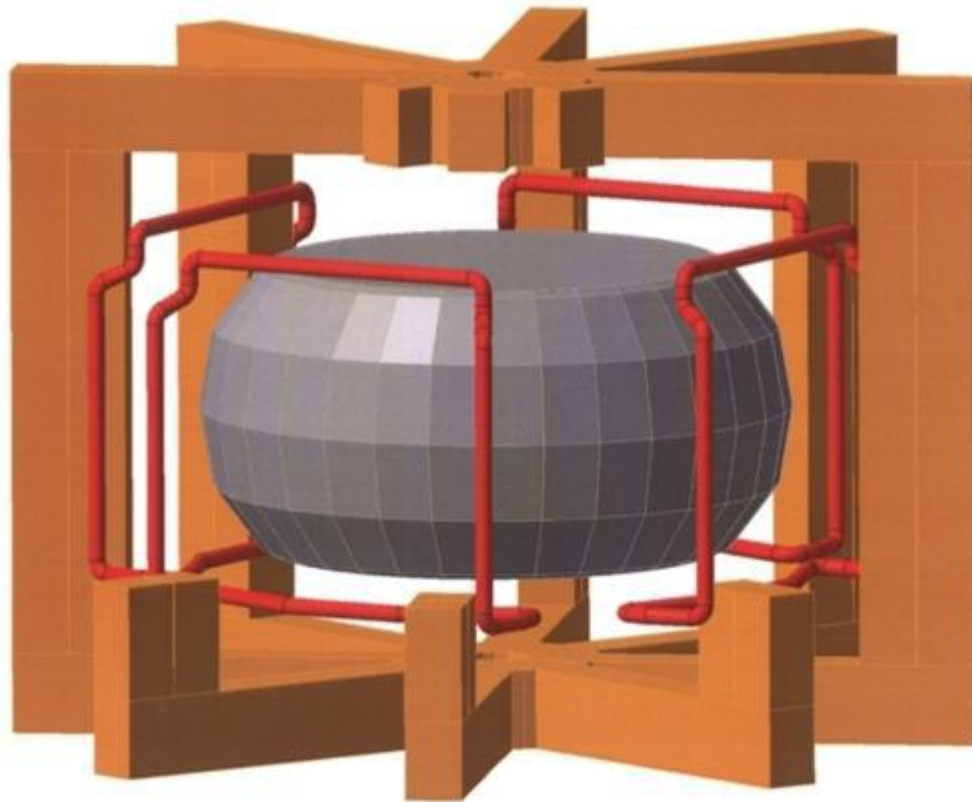
vacuum, $n = 2$



plasma, $n = 1$



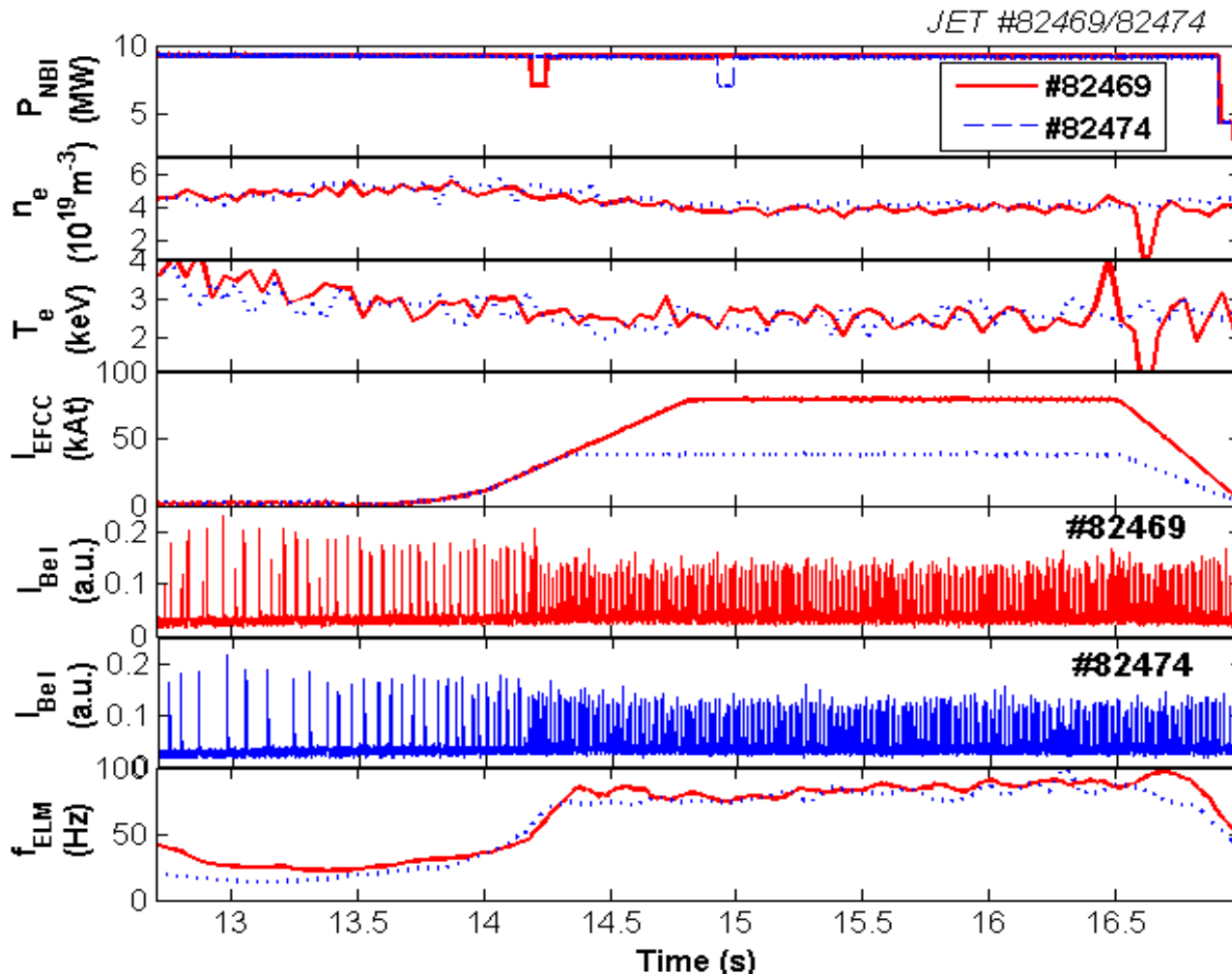
plasma, $n = 2$



On JET, 14 saddle loops are fitted to the external wall of the each octant of the vacuum vessel and cover basically the whole surface area of the vessel

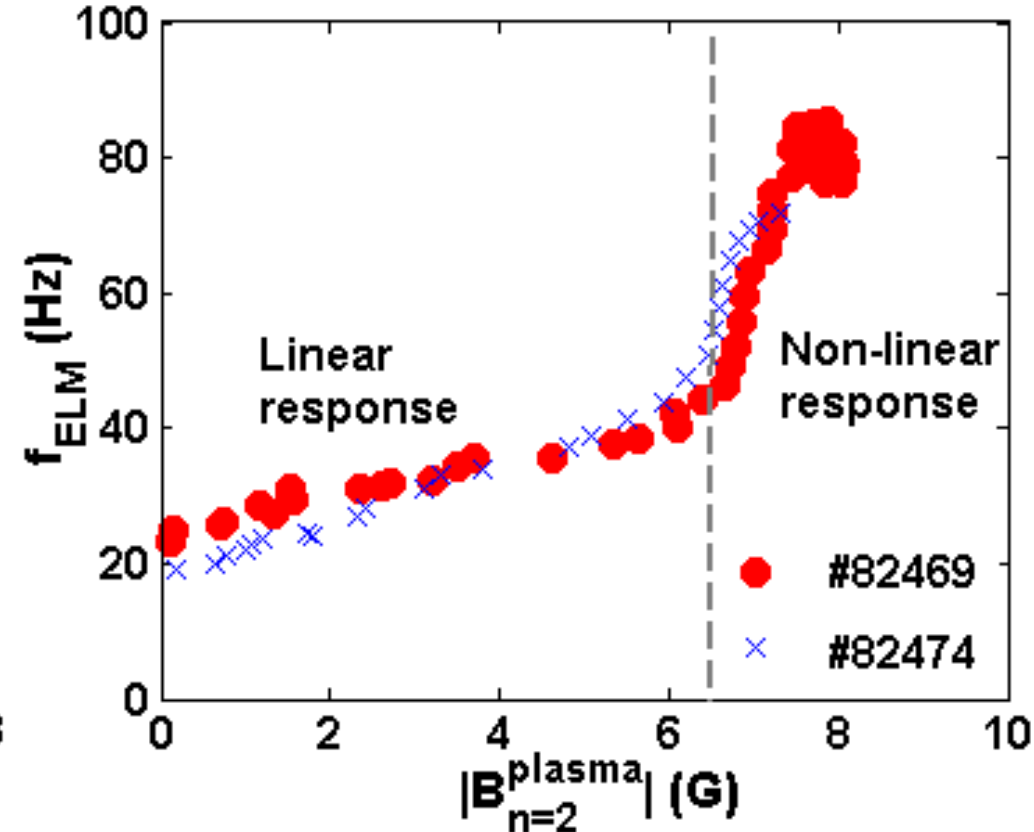
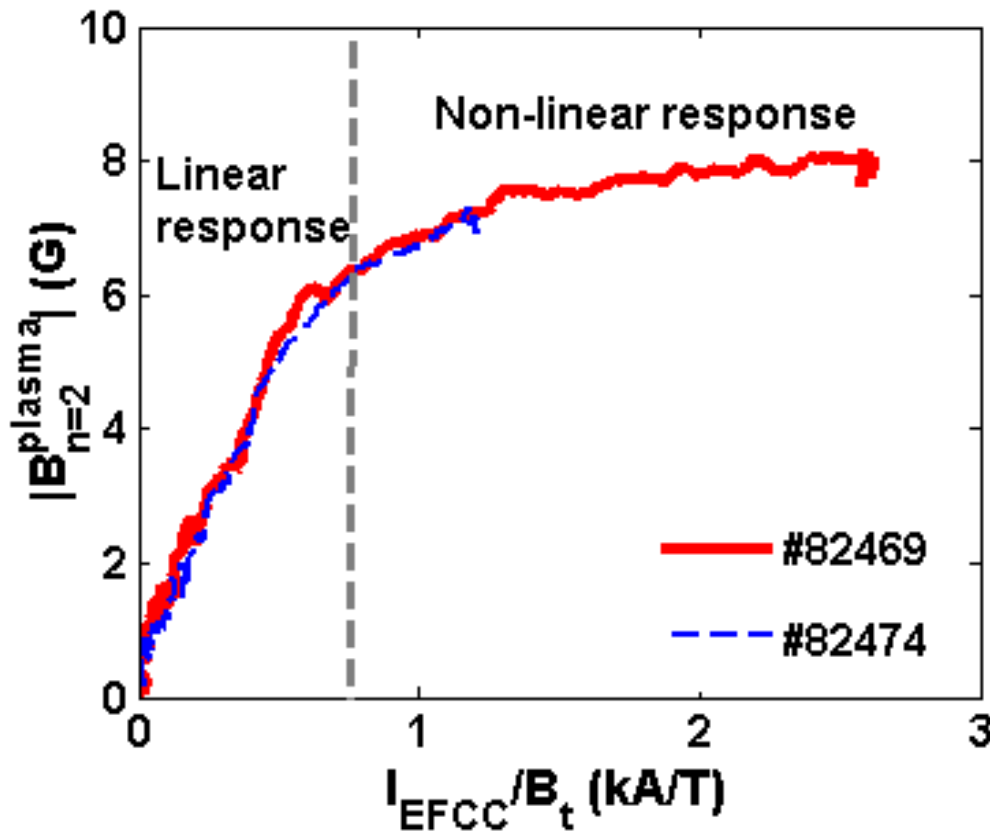


ELM Mitigation with $n = 2$ Field in Low Collisionality Plasmas on JET with the ILW



- $B_t=1.85\text{T}$; $I_p=1.4\text{MA}$
- There is a threshold of I_{EFCC} for ELM mitigation
- f_{ELM} increases from 20Hz to 80Hz when $I_{EFCC}>2.4 \text{ kA}$.
- No further increase f_{ELM} even increasing I_{EFCC} further up to 5.0 kA.

f_{ELM} vs plasma response to $n=2$ field

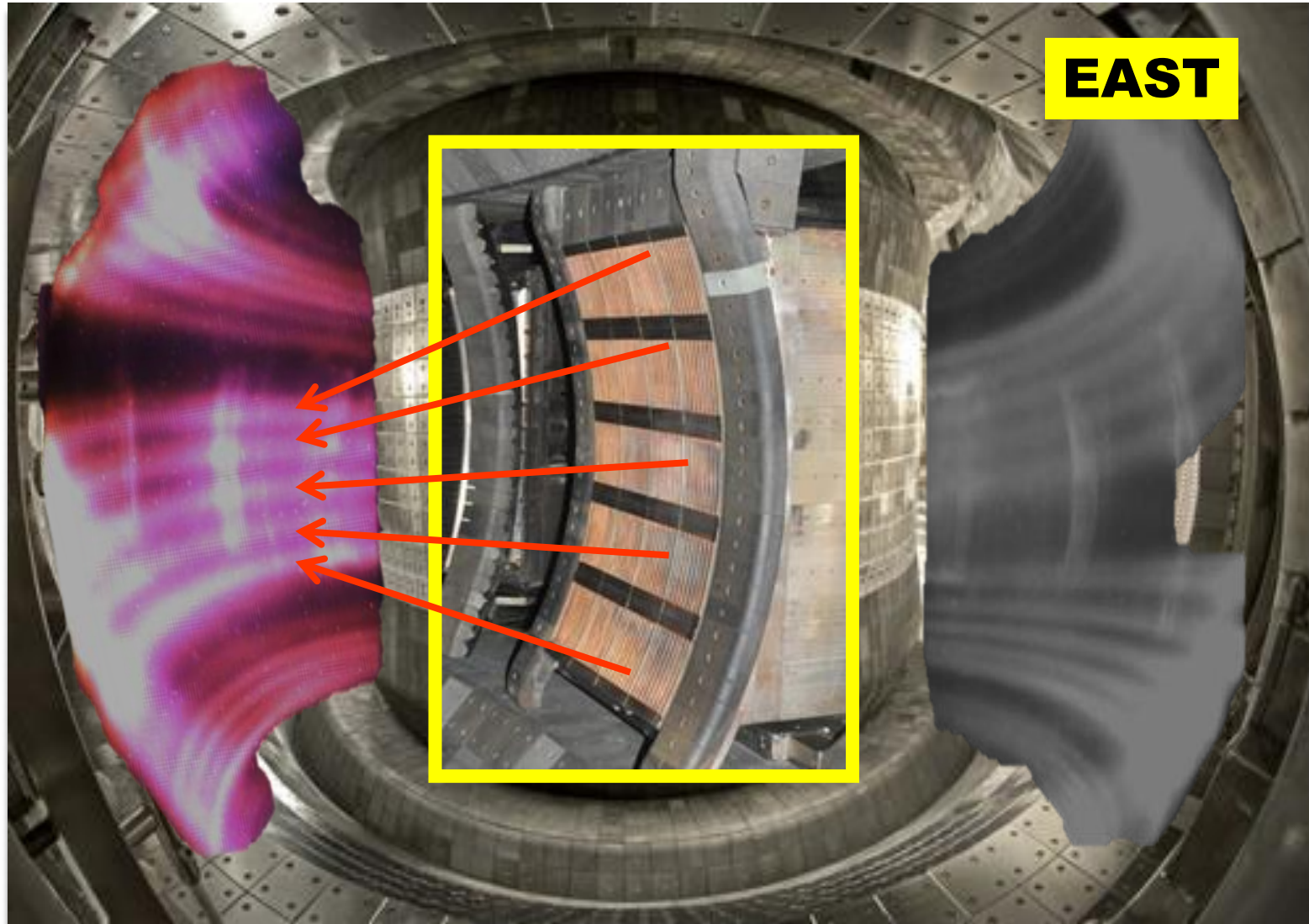


f_{ELM} does not change once the plasma response saturated.

Y. Liang, et al., IAEA, 2012

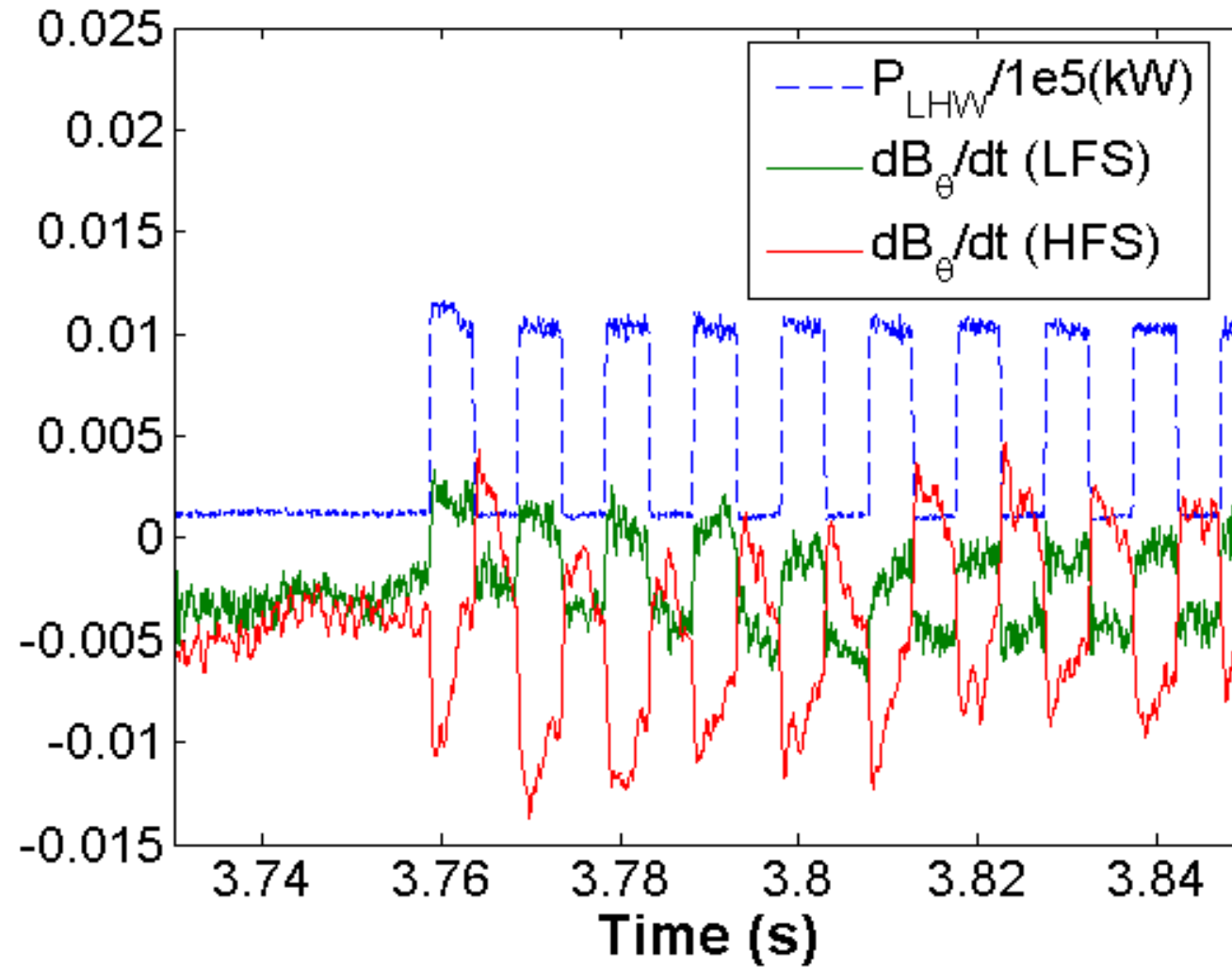


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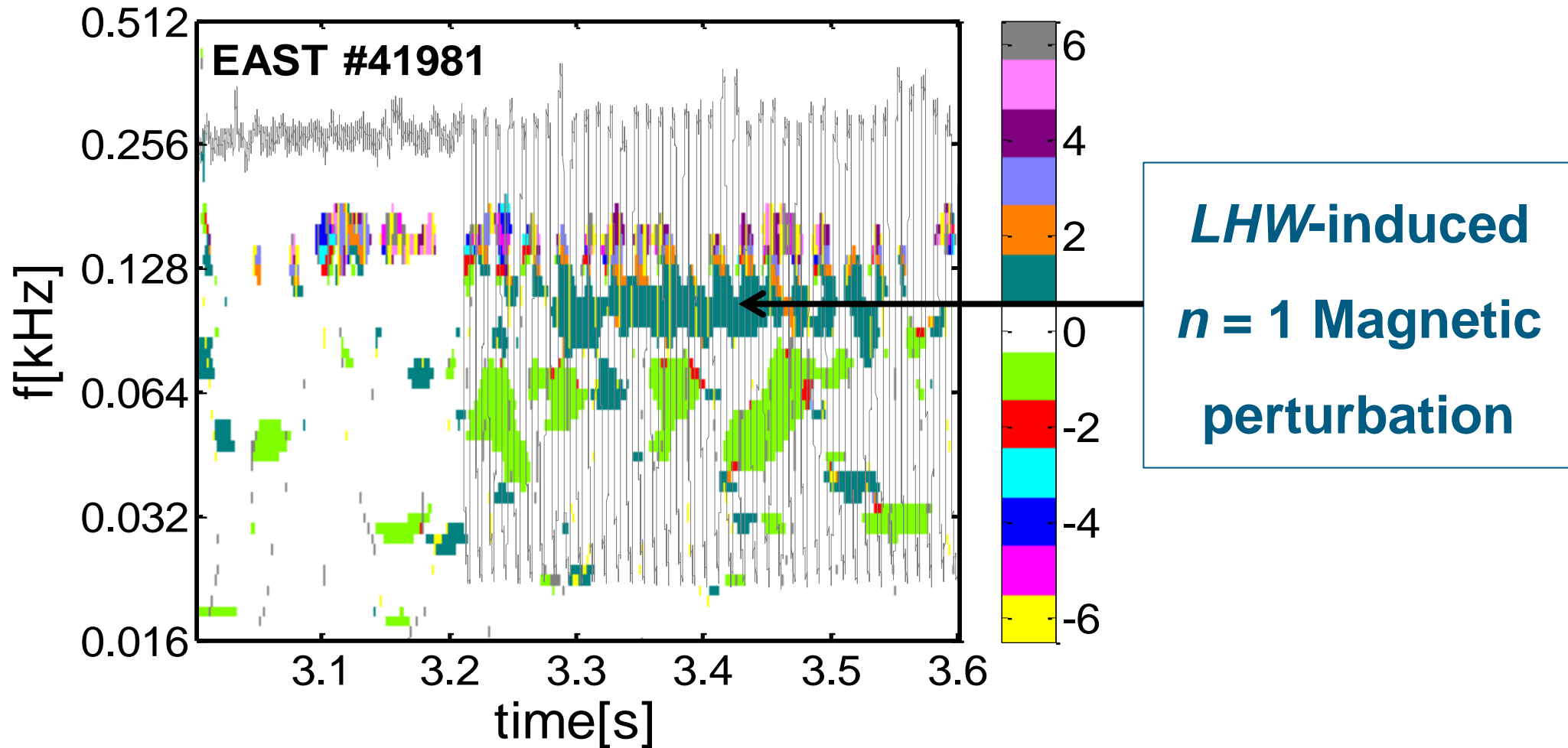
Y. Liang, et al., IAEA, 2012

Helical Current Filaments (HCF_s) Induced by LHCD on EAST



Modulation of P_{LHCD} ; $f = 100$ Hz; $P_{LHCD} = 0-1.2$ MW

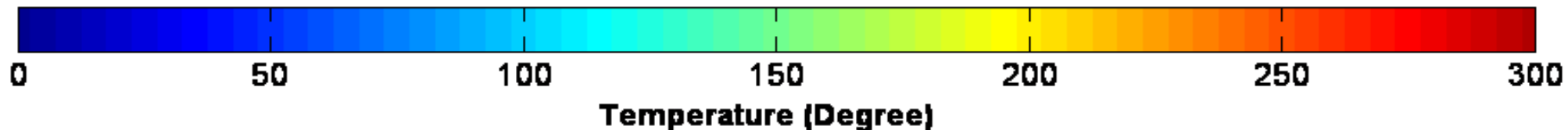
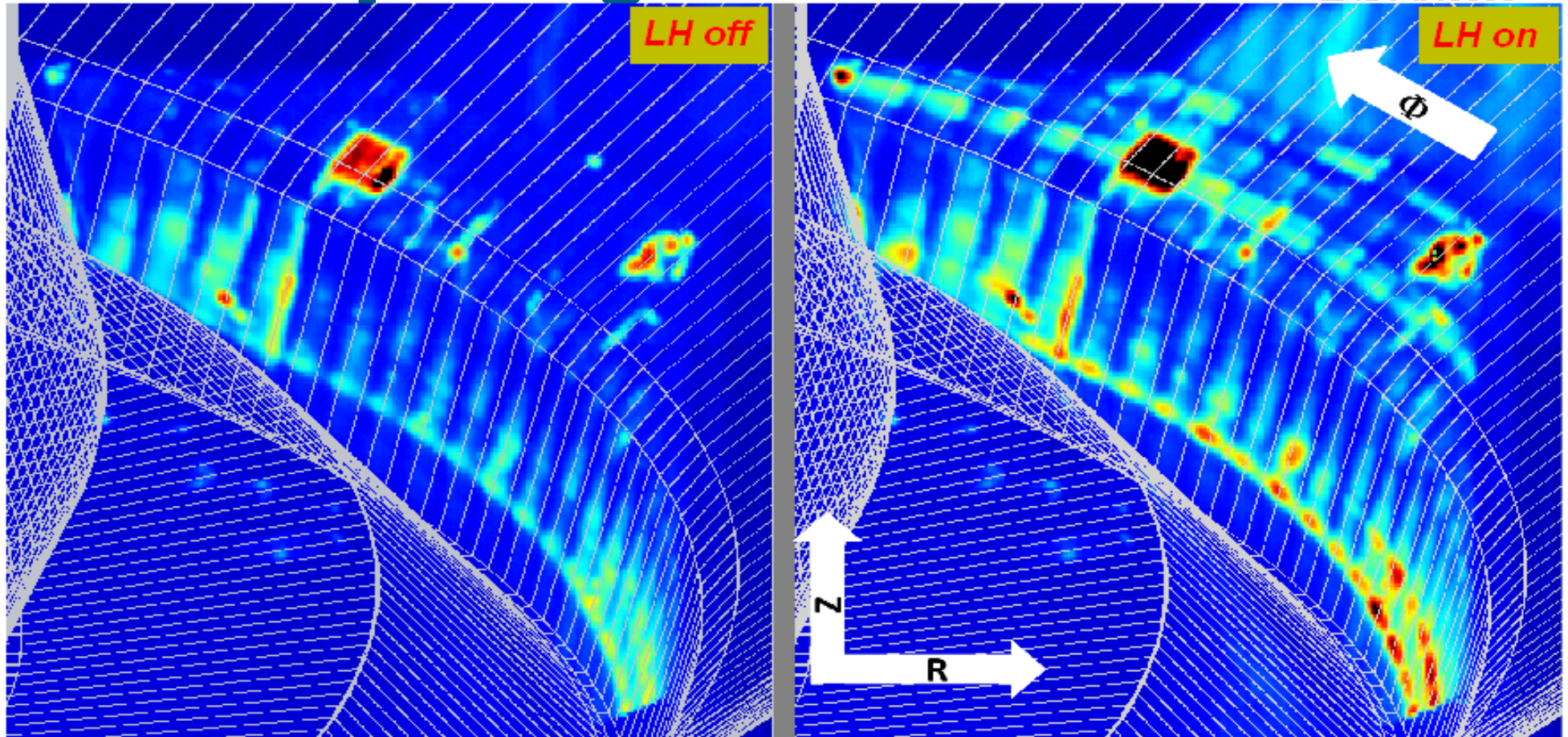
Helical Current Filaments (HCF_s) Induced by LHCD on EAST



Modulation of P_{LHCD} ; $f = 100$ Hz; $P_{LHCD} = 0-1.2$ MW

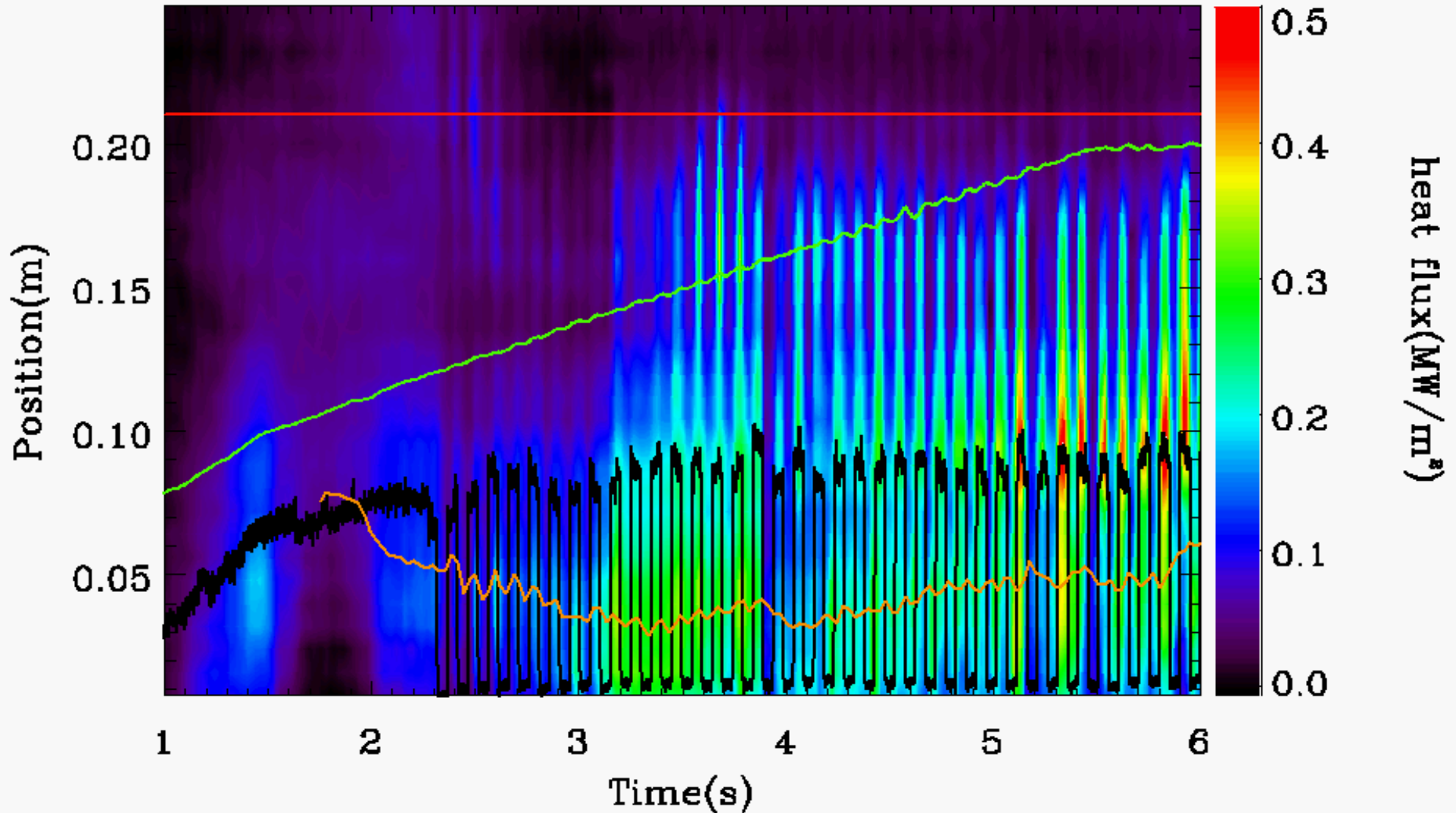
LHW induced Footprint Structures: Splitting of Strike Points

EAST #43380

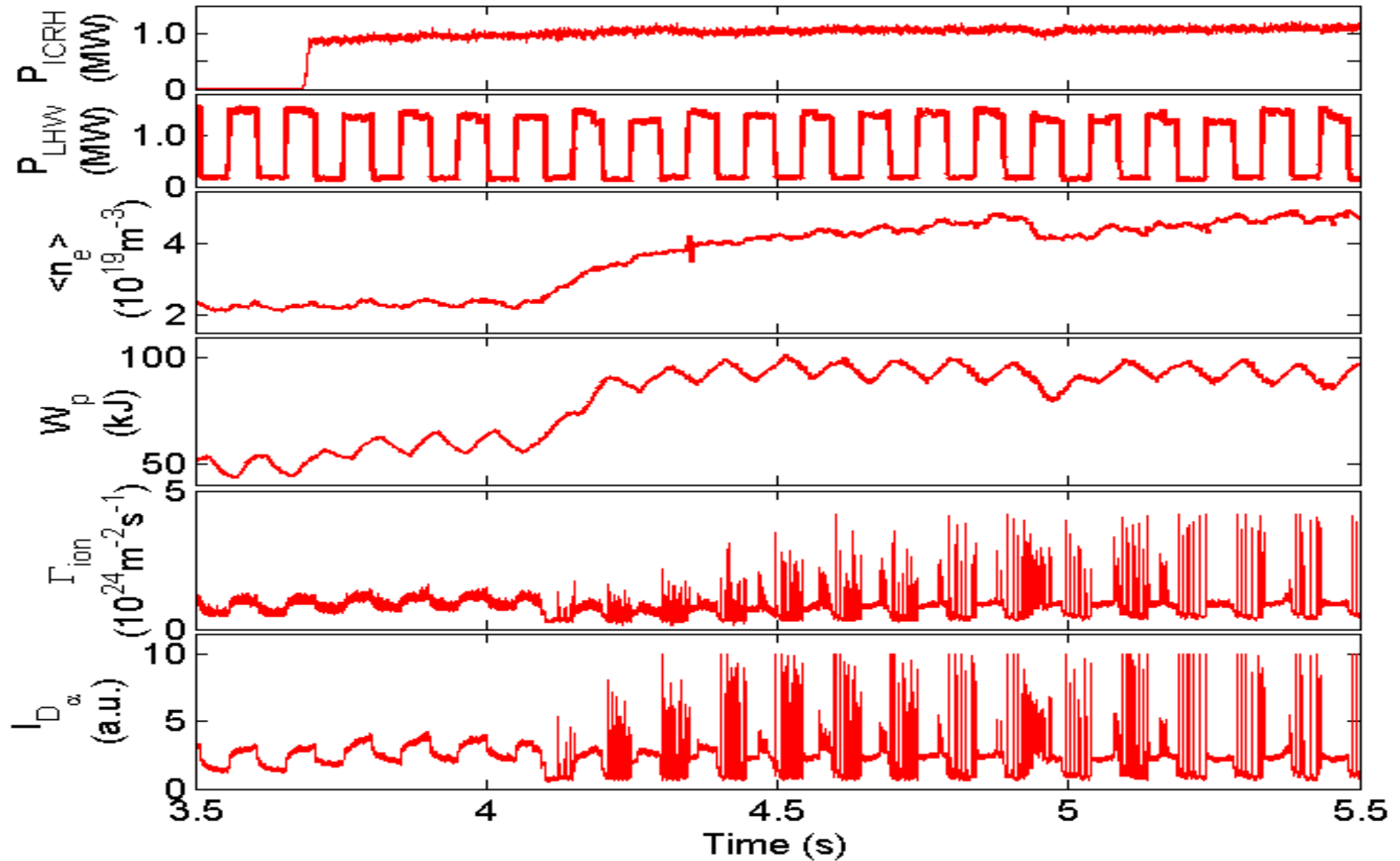


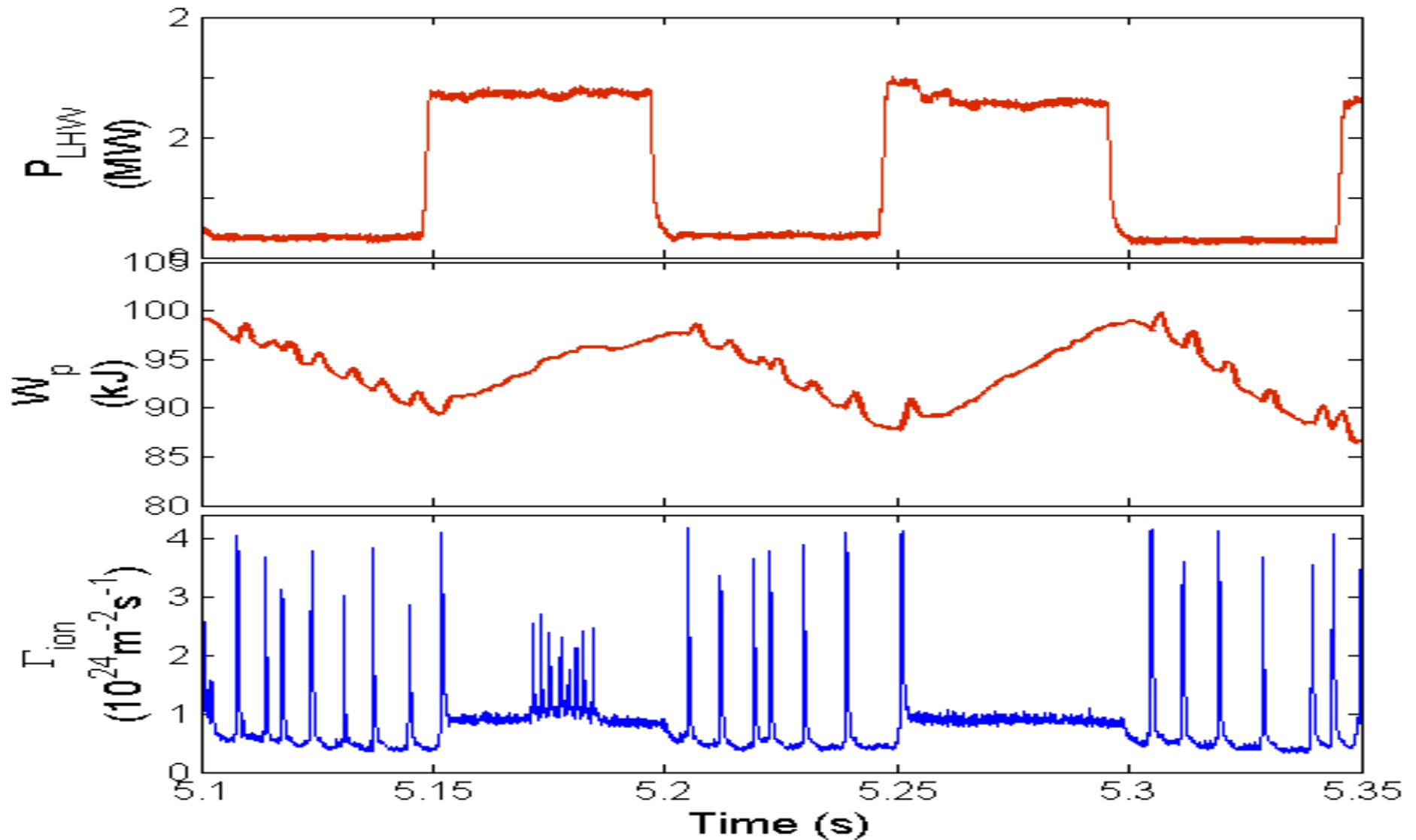
Footprint Structures: Edge Safety Factor Dependence

#42327(600KA) I_p LHW OSP Boundary of low outer plate

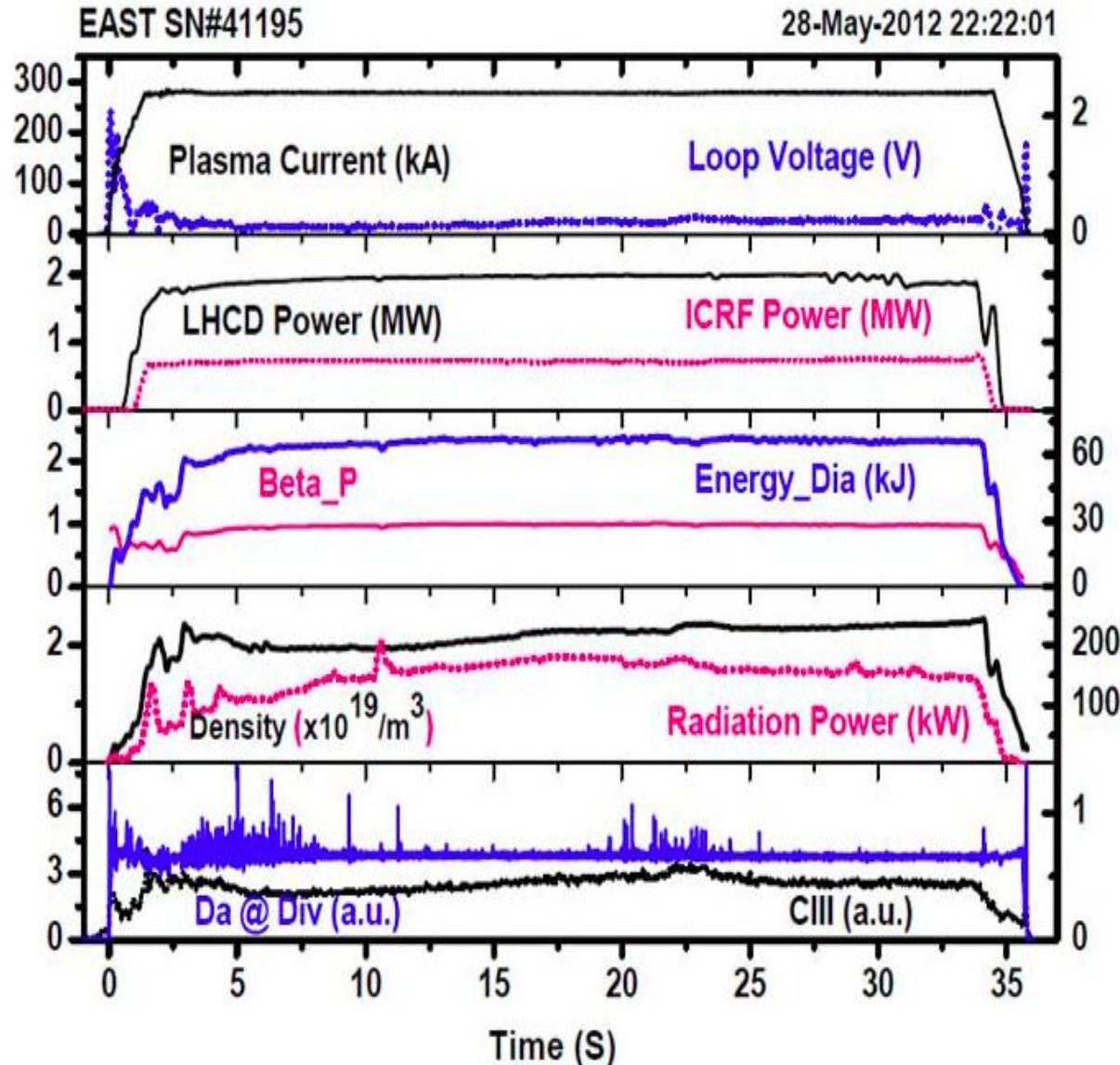


EAST #41985





Over 30s H-mode Operation



□ H-mode duration up to 32s with H98~0.9.

□ Small ELMs with LHCD.

□ Control density at $\sim 2 \times 10^{19}/\text{m}^3$ for LHCD.



Summary

- 3D magnetic perturbation has been widely applied for active control of edge transport and MHD instabilities.
- The plasma response may modify the applied RMP field substantially, possibly leading to elimination of magnetic islands and the edge stochastic region that the vacuum models predict.
- Further investigation of plasma responses to the RMP is needed for a reliable application of the RMP for plasma transport and stability control on future fusion devices, i.e. ITER.
- On EAST, LHW appears to induce a profound change in the edge magnetic topology and ELMs. It is important information for the on-going discussion of ITER LHW system.

Rotation Dependence of Field Penetration in Core Plasma

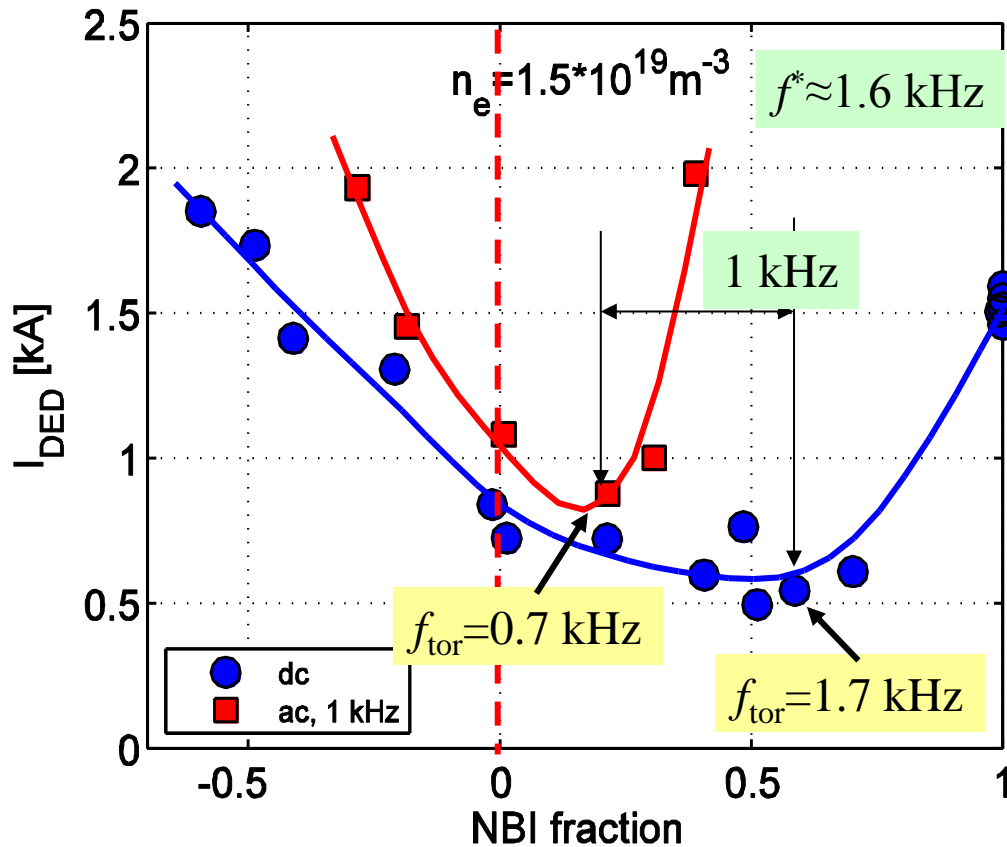
Field penetration (formation of islands) in the core plasma is a well studied problem

(Fitzpatrick theory)

$$V_{\perp,e} = V_{ExB} + V_e^*$$

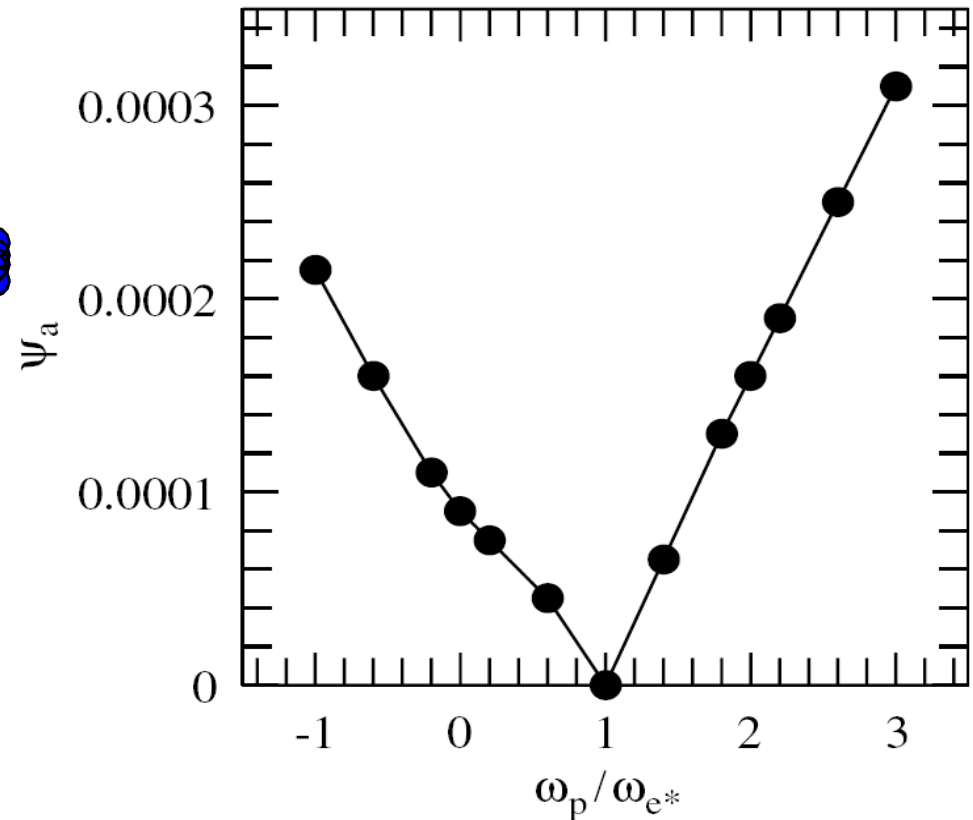
$$f_{MHD} = f_{DED}$$

TEXTOR Experiment



HR Koslowski, et al., Nucl. Fusion **46** (2006) L1–L5

Numerical nonlinear two fluids MHD Modeling



Q Yu, et al., Nucl. Fusion **48** (2008) 024007